

August 21, 1987

Docket No. 50-247

Mr. Murray Selman
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
New York, New York 10511

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Dear Mr. Selman:

The Commission has issued the enclosed Amendment No. 121 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated December 8, 1986.

The amendment revises the surveillance requirement contained in the Technical Specifications for testing the partial movement of control rods. The proposed change clarifies the term "partial movement." In addition, it changes the testing frequency from a nominal two week surveillance interval to a nominal 31 day interval.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No. 121 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

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Mr. Murray Selman
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

cc:

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, New York 10511

Mr. Jay Dunkleberger
Office of Policy Analysis
and Planning
York State Energy Office
Building 2, Empire State Plaza
Albany, New York 12223

Robert L. Spring
Manager of Regulatory Affairs
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 38
Buchanan, New York 10511

Brent L. Brandenburg
Assistant General Counsel
Consolidated Edison Company
of New York, Inc.
4 Irving Place - 1822
New York, New York 10003

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Carl R. D'Alvia, Esquire
Attorney for the Village of
Buchanan, New York
395 South Riverside Avenue
Croton-on-Hudson, New York 10520

Director, Technical Development
Programs
State of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Mr. Peter Kokolakis, Director
Nuclear Licensing
New York Power Authority
123 Main Street
White Plains, New York 10601

Mr. Walter Stein
Secretary - NFSC
Consolidated Edison Company
of New York, Inc.
4 Irving Place - 1822
New York, New York 10003

Ezra I. Bialik
Assistant Attorney General
Environmental Protection Bureau
New York State Department of Law
2 World Trade Center
New York, New York 10047



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated December 8, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 121, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

for Joseph D. Neighles
Robert A. Capra, Acting Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 21, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

4.1-1

Table 4.1-3

Insert Pages

4.1-1

Table 4.1-3

4. Surveillance Requirements

4.1 Operational Safety Review

Applicability

Applies to items directly related to safety limits and limiting conditions for operation.

Objective

To specify the minimum frequency and type of surveillance to be applied to plant equipment and conditions.

Specification

- a. Calibration, testing and checking of analog channels, and testing of logic channels shall be performed as specified in Table 4.1-1.
- b. Sampling and equipment tests shall be conducted as specified in Tables 4.1-2 and 4.1-3, respectively.
- c. Performance of any surveillance test outlined in these specifications is not immediately required if the plant condition is the same as the condition into which the plant would be placed by an unsatisfactory result of that test. Such tests will be performed before the plant is removed from the subject condition that has precluded the immediate need to run the test. If the test provisions require that a minimum higher system condition must first be established, the test will be performed promptly upon achieving this minimum condition. The following surveillance tests, however, must be performed without the above exception:

- | | |
|---------------|--------------------|
| o Table 4.1-1 | Items 3 and 19 |
| o Table 4.1-2 | Items 1, 2, and 10 |
| o Table 4.1-3 | Items 2, 6 |

Basis

A surveillance test is intended to identify conditions in a plant that would lead to a degradation of reactor safety. Should a test reveal such a condition, the Technical Specifications require that either immediately, or after a specified period of time, the plant be placed in a condition which mitigates or eliminates the consequences of additional related casualties or accidents. If the plant is already in a condition which satisfies the failure criteria of the test, then plant safety is not compromised and performance of the test yields information that is not necessary to determine safety limits or limiting conditions for operation of the plant. The surveillance test need not be performed, therefore, as long as the plant remains in this condition. However, this surveillance test should be performed prior to removing the plant from the subject condition that has precluded the immediate need to run the

TABLE 4.1-3 (1 of 1)

FREQUENCIES FOR EQUIPMENT TESTS

	<u>Check</u>	<u>Frequency</u>	<u>Maximum Time Between Tests</u>
1. Control Rods	Rod drop times of all control rods	Each refueling shutdown	**
2. Control Rods	Movement of at least 10 steps in any one direction of all control rods	Every 31 days during reactor critical operations	**
3. Pressurizer Safety Valves	Set point	Each refueling shutdown	**
4. Main Steam Safety Valves	Set point	Each refueling shutdown	**
5. Containment Isolation System	Automatic Actuation	Each refueling shutdown	**
6. Refueling System Interlocks	Functioning	Each refueling shutdown prior to refueling operation	Not Applicable
7. Diesel Fuel Supply	Fuel Inventory	Weekly	10 days
8. Turbine Steam Stop, Control Valves	Closure	Monthly****	45 days****
9. Cable Tunnel Ventilation Fans	Functioning	Monthly	45 days

**See Specification 1.9.

****This test may be waived during end-of-cycle operation when reactor coolant boron concentration is equal to or less than 150 ppm, due to operational limitations.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

INTRODUCTION

By letter dated December 8, 1987, Consolidated Edison Company of New York requested changes to the Technical Specifications for the Indian Point Nuclear Generating Unit No. 2 (IP-2). This change would revise the surveillance requirements for testing all control rods for partial movement. In addition, it would change the testing frequency to once every 31 days. Some purely administrative changes to achieve consistency throughout the Technical Specifications were also requested. The changes are editorial in nature.

DISCUSSION AND EVALUATION

The present Technical Specifications require surveillance tests of all control rods for "partial movement", but do not specify the amount of movement. The proposed change would specify control rod movement of at least 10 steps in any one direction, thus clarifying the minimum number of steps required to assure control rod freedom of movement.

The proposed change from a nominal two week surveillance interval to a nominal 31 day surveillance interval would reduce the mechanical wear on the drive mechanisms and reduce the wear on rod control cluster cladding caused by more frequent rod insertions.

We have reviewed the licensee's submittal and agree that specifying the minimum number of steps required in order to assure control rod freedom of movement is desirable. This change will enhance the clarity of the Technical Specification, decrease the likelihood of misunderstanding and ensure that the rods are moved an adequate amount.

Since initial plant startup of Indian Point Unit No. 2, the licensee has never found an immovable control rod during movement exercise. Never has a control rod failed to go to the fully inserted position when required. In addition, the staff concludes that the change in surveillance testing frequency is not likely to have a significant impact on plant safety from an overall risk assessment standpoint. Based on this, we agree that changing from a two week to the 31 day surveillance interval remains conservative. In addition, these changes are consistent with the Standard Technical Specifications for Westinghouse Pressurized Water Reactors. The administrative changes are editorial in nature and have no impact on safety. Therefore, we find the proposed changes to be acceptable.

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ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

PRINCIPAL CONTRIBUTOR:

M. Chatterton

Dated: August 21, 1987

August 21, 1987

MEMORANDUM FOR: Sholly Coordinator

FROM: Robert A. Capra, Acting Director
Project Directorate I-1
Division of Reactor Projects, I/II

SUBJECT: REQUEST FOR PUBLICATION IN BI-WEEKLY FR NOTICE - NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

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C. Vogan
M. Slosson
J. Scinto

Consolidated Edison Company of New York, Docket No. 50-247, Indian Point Nuclear Generating Unit No. 2, Westchester County, New York

Date of application for amendment: December 8, 1986

Brief description of amendment: The amendment revises the surveillance requirement contained in the Technical Specifications for testing the partial movement of control rods. The proposed change clarifies the term "partial movement." In addition, it changes the testing frequency from a nominal two week surveillance interval to a nominal 31 day interval.

Date of issuance: August 21, 1987

Effective date: August 21, 1987

Amendment No.: 121

Facilities Operating License No. DPR-26: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: March 25, 1987 (52 FR 9565)

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated August 21, 1987

No significant hazards consideration comments received: No

Local Public Document Room location: White Plains Public Library, 100 Martine Avenue, White Plains, New York, 10610.

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Robert A. Capra, Acting Director
Project Directorate I-1
Division of Reactor Projects, I/II

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MSlosson
6/10/87

*Dr. PGC
Capra
7/87*

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