

September 1, 1987

Docket No. 50-247

Mr. Murray Selman
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

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E. Jordan
J. Partlow
T. Barnhart(4)
Wanda Jones
E. Butcher
ACRS(10)
ARM/LFMB

Dear Mr. Selman:

The Commission has issued the enclosed Amendment No. 122 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated May 29, 1987 (TAC 65626).

The amendment revises the Technical Specifications to permit a one-time extension of the surveillance interval limits for various systems and components. Specifically the Technical Specifications are modified to extend the 3.25 total time interval limit over three consecutive surveillance intervals to allow testing to be performed during the scheduled 1987 refueling/maintenance outage rather than requiring a special plant shutdown solely to perform these tests.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

- 1. Amendment No. 122 to DPR-26
- 2. Safety Evaluation

cc: w/enclosures
See next page

* SEE PREVIOUS CONCURRENCE

PDI-1
CVogan*
8/13/87

PDI-1
MSlosson*
8/13/87

OGC
MKarman*
8/18/87

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RCapra
9/1/87

SICB
JJoyce*
8/26/87

RAC

SRXB
RJones*
8/31/87

SELB
FRosa*
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OTSB
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PDI-1 *CV*
CVogan
8/15/87

PDI-1 *MS*
MSlosson
8/13/87

OGC
M. Harman
8/18/87

PDI-1
RCapra
8/1/87

SPLB
J. Jorgce
8/26/87

SRXB
R.C. Jones
8/31/87

SELB
E. ROSA
7/29/87

SPLB
J. WERMIEZ
7/1/87

OTSB
R. ENCH
7/2/87
per 7/2/87
note
MS

per 7/29/87
MEMO
MS
per 7/1/87
Note
MS

Mr. Murray Selman
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of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 122
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated May 29, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 122, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Acting Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 1, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 122 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Page

1-4

Insert Page

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1.8 Quadrant Power Tilt Ratio

The quadrant power tilt ratio shall be the ratio of the maximum upper excore detector calibrated output to the average of the upper detector calibrated outputs, or the ratio of the maximum lower excore detector calibrated output to the average of the lower excore detector calibrated outputs, whichever is greater. With one excore detector inoperable, the remaining three detectors shall be used for computing the average.

1.9 Surveillance Intervals

Unless otherwise noted in an individual surveillance requirement, surveillance intervals shall be as specified in Table 1-1 with extensions as provided in 1.10 below. The extensions provided in 1.10 below also apply to surveillance intervals not listed in Table 1-1 unless the extensions are specifically not allowed.

1.10 Surveillance Interval Maximums

Each Surveillance Requirement shall be performed within the specified time interval with:

- a. A maximum allowable extension not to exceed 25% of the surveillance interval, and

A total maximum combined interval time for any 3 consecutive surveillance intervals not to exceed 3.25 times the specified surveillance interval.*

1.11 PRESSURE BOUNDARY LEAKAGE

PRESSURE BOUNDARY LEAKAGE shall be leakage (except steam generator tube leakage) through a non-isolatable fault in a Reactor Coolant System component body, pipe wall or vessel wall.

1.12 IDENTIFIED LEAKAGE

IDENTIFIED LEAKAGE SHALL BE:

- a. Reactor coolant system leakage into closed systems such as pump seal or valve packing leaks that are captured and conducted to a collecting tank, or
- b. Reactor coolant system leakage through a steam generator to the secondary system, or
- c. Reactor coolant system leakage through the RCS/RHR pressure isolation valves, or

* There shall be an exemption for surveillance requirements listed in Table 1 of the letter from Murray Selman to Document Control Desk dated May 29, 1987. The 3.25 maximum combined interval may be extended to permit tests and calibrations to be performed prior to startup from the Cycle 8/9 refueling outage.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555



SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 122 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

INTRODUCTION

By letter dated May 29, 1987 Consolidated Edison Company of New York, Inc. (the licensee) requested a modification to the Indian Point Nuclear Generating Unit No. 2 (IP-2) Technical Specifications to permit a one-time extension of the surveillance interval limits for various systems and components shown in Table 1. The licensee requested that the Technical Specifications be modified to extend the 3.25 total time interval limit over three consecutive surveillance intervals to allow testing to be performed during the scheduled 1987 refueling/maintenance outage rather than requiring a special plant shutdown solely to perform these tests. The earliest surveillance test would have to be performed in September 1987. According to the licensee's submittal, the 1987 refueling outage is scheduled to commence in November 1987.

DISCUSSION AND EVALUATION

The IP-2 Technical Specification 1.10 states that each surveillance requirement shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the surveillance interval and with a total maximum combined interval time for any 3 consecutive surveillance intervals not to exceed 3.25 times the specified surveillance interval. This is consistent with the Westinghouse Standard Technical Specifications. There are various components and systems for which the surveillance interval is given in the Technical Specifications as each refueling. A refueling interval is defined in the Technical Specifications as 18 months.

Generic Letter 83-27 dated July 6, 1983 entitled "Surveillance Intervals in Standard Technical Specifications" indicates that the 18 month surveillance interval is based on reactor operating experience and the recognition of reactors utilizing 18 month fuel cycles. The basis for the provision which allows any surveillance interval to be extended by 25% is to provide the necessary operational flexibility which may be required due to scheduling and operational performance considerations. Generic Letter 83-27 also indicates that one time changes may be granted for plant specific conditions where adequate justification is provided.

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During the last four consecutive fuel cycles, IP-2 incurred two extended outages, due to unplanned events and long fuel cycles, because of occasional reactor shutdowns and extended low power operation. As a result, for some equipment (Table 1) normally tested at an 18 month interval during refueling, the maximum combined interval of 3.25 times the specified surveillance will be reached before the next scheduled refueling outage. Without a one time extension to exceed the 3.25 criteria, IP-2 would be required to shutdown approximately two months prior to their planned refueling/maintenance outage.

The licensee has indicated that the earliest that surveillance would be required is during September 1987. The next IP-2 refueling outage is scheduled for the beginning of November 1987. Therefore, the maximum extension time is less than 2 months. This represents an increase of 3% from the 3.25 criteria. However, Consolidated Edison has indicated that even with the extension, all surveillance tests for the equipment in Table 1 would be performed within the allowable existing permissible Technical Specification interval between any two tests, e.g., 18 months plus 25%.

The licensee has reviewed the results of previous surveillance tests and concluded that there is no reason to expect significant safety-related component failures during the extended surveillance interval.

The staff concludes that the quality of the components listed in Table 1 and its ability to perform will be maintained during the extension period to at least the equivalent of that level currently provided by the Technical Specifications for a maximum surveillance interval (i.e., 18 months plus 25%). Furthermore, the staff concludes that extension of 3% is insignificant with regard to the surveillance interval and does not warrant an additional plant shutdown.

With regard to future calculations for the 3.25 criteria, this extension can be disregarded. However, it should be noted that although we are granting this one-time extension, the licensee should plan future surveillance in order that such extensions are not necessary.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

PRINCIPAL CONTRIBUTOR:

M. Slosson

Dated: September 1, 1987

TABLE 1

SURVEILLANCE INFORMATION IN SUPPORT
OF PROPOSED TECHNICAL SPECIFICATION
CHANGE ON MAXIMUM SURVEILLANCE INTERVALS

TEST NO.	TEST DESCRIPTION	TECH SPEC REFERENCE
PC-R1A	RCS LOOPS HOT AND COLD LEGS NARROW RANGE TEMPERATURE	Table 4.1-1 Item 4
PC-R3	PRESSURIZER LEVEL	Table 4.1-1 Item 6
PC-R4	PRESSURIZER PRESSURE	Table 4.1-1 Item 7
PC-R5A	6.9 KV UNDERVOLTAGE AND RESPONSE TIME	Table 4.1-1 Item 8
PC-R7	STEAM GENERATOR LEVEL	Table 4.1-1 Item 11
PC-R13	VOLUME CONTROL TANK LEVEL	Table 4.1-1 Item 17
PC-R15B	AREA RADIATION MONITOR	Table 4.1-1 Item 19
PC-R17E	ACCUMULATOR PRESSURE	Table 4.1-1 Item 22
PC-R18	STEAM LINE PRESSURE	Table 4.1-1 Item 22
PC-R20	INDEPENDENT ELECTRICAL OVERSPEED PROTECTION SYSTEM	Table 4.1-1 Item 24
PT-R5A	HOT SETTING OF PRESSURIZER SAFETY VALVES BY WYLE LABS	Table 4.1-3 Item 26
PT-R10	FAN COOLER UNITS FILTRATION	Section 4.5.D.2
PT-R40	FCU WATER SPRAY AND ALARM SYSTEM	Section 4.14.B
PT-R47	V.C. SMOKE DETECTOR OPERABILITY	Section 4.14.D
PT-R61	480 VOLT BREAKER UNDERVOLTAGE RELAYS	Table 4.1-1 Item 29
PI-V1A	SHOCK SUPPRESSOR (SNUBBER) VISUAL INSPECTION	Section 4.12.A