

October 14, 1987

Docket No. 50-247

Mr. Murray Selman
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

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Dear Mr. Selman:

The Commission has issued the enclosed Amendment No. 124 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated May 20, 1987 (TAC 65625).

The amendment revises the Technical Specifications relating to the limiting conditions for operation of the Auxiliary Feedwater Pumps to be consistent with the Standard Technical Specifications. The change adds limiting conditions for operation for up to three Auxiliary Feedwater Pumps that may be in an inoperable condition.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No. 124 to DPR-26
2. Safety Evaluation

cc: w/enclosures

See next page

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CVogan *w*
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Indian Point Nuclear Generating
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 124
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated May 20, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.124 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Acting Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 14, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 124 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

3.4-1

3.4-1(b)

Insert Pages

3.4-1

3.4-1(b)

3.4 STEAM AND POWER CONVERSION SYSTEM

Applicability

Applies to the operating status of the Steam and Power Conversion System.

Objective

To define conditions of the turbine cycle steam-relieving capacity. Auxiliary Feedwater System and City Water System operation is necessary to ensure the capability to remove decay heat from the core.

Specification

- A. The reactor shall not be heated above 350°F unless the following conditions are met:
- (1) A minimum ASME code approved steam relieving capability of twenty (20) main steam valves shall be operable (except for testing). With up to three (per steam generator) of the twenty main steam line code-approved safety relief valves inoperable, heat-up above 350°F and power operation is permissible provided either the inoperable valve(s) is restored to operable status or the Power Range Neutron Flux High Trip Setpoint is reduced per Table 3.4-1.
 - (2) Three auxiliary feedwater pumps, each capable of pumping a minimum of 400 gpm, must be operable.
 - (3) A minimum of 360,000 gallons of water in the condensate storage tank and a backup supply from the city water supply.
 - (4) Required system piping, valves, and instrumentation directly associated with the above components operable.
 - (5) The main steam stop valves are operable and capable of closing in five seconds or less.
 - (6) The total iodine activity of I-131 and I-133 on the secondary side of the steam generator shall be less than or equal to 0.15 uCi/cc.

B. Except as modified by 3.4.B(1) and 3.4.C below, if any of the conditions of 3.4.A above cannot be met within 72 hours, the reactor shall be placed in the hot shutdown condition within the next 12 hours and subsequently cooled below 350°F using normal operating procedures.

(1) With one or more auxiliary feedwater pump(s) inoperable take the following actions:

a) With one auxiliary feedwater pump inoperable, restore the pump to operable status within 72 hours or place the reactor in the hot shutdown condition and subsequently cool the RCS to below 350°F using normal operating procedures within the next 12 hours.

b) With two auxiliary feedwater pumps inoperable, place the reactor in hot shutdown and subsequently cool the RCS below 350°F using normal operating procedures within 12 hours.

c) With three auxiliary feedwater pumps inoperable, immediately initiate corrective action to restore at least one auxiliary feedwater pump to operable status while maintaining power at existing level until at least one auxiliary feedwater pump has been restored to operable status, and then immediately comply with the requirements of 3.4.B(1)(b) above.

C. If when above 350°F one of the series valves (CT-6 and/or CT-64) in the condensate storage tank discharge line is closed, then:

(1) Immediately place the auxiliary feedwater pump controls in the manual mode, and

(2) Within one (1) hour, either valve(s) shall be reopened or the valves from the alternate city water supply shall be opened and the auxiliary feedwater pump controls restored to the automatic mode.

If these requirements cannot be met, then:

(1) maintain the plant in safe stable mode which minimizes the potential for a reactor trip, and

(2) continue efforts to restore water supply to the auxiliary feedwater system, and

(3) notify the NRC within 24 hours regarding the planned corrective action.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 124 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

INTRODUCTION

By letter dated May 20, 1987, Consolidated Edison Company of New York, Inc. (the licensee) submitted a proposed amendment to the Indian Point 2 Technical Specifications adding Limiting Conditions of Operation (LCO) for inoperable auxiliary feedwater pumps.

DISCUSSION AND EVALUATION

The current Indian Point 2 Technical Specifications require that three auxiliary feedwater pumps be operable when the Reactor Coolant System is above 350°F. If this condition can not be met within 72 hours the Technical Specifications require that within 12 hours the reactor be placed in the hot shutdown condition and the Reactor Coolant System be cooled to below 350°F. The current Technical Specifications do not distinguish between the condition of one, two or three auxiliary feedwater pumps inoperable.

The proposed Technical Specifications will increase the limitations imposed by the Technical Specifications. The conditions imposed if one auxiliary feedwater pump is inoperable will remain the same as the current Technical Specification limits. With two auxiliary feedwater pumps inoperable, the LCO will be revised to require that the reactor be placed in hot shutdown and the reactor cooled below 350°F, both within 12 hours. With three auxiliary feedwater pumps inoperable, the Technical Specifications will be revised to require that corrective action to restore at least one auxiliary feedwater pump to operable status be initiated as soon as possible. Once restored, the requirements for two inoperable pumps are applied.

The staff finds the proposed Technical Specifications to be conservative and to add restrictions not currently in the Technical Specifications. The proposed Technical Specifications are also in accordance with Standard Technical Specifications. The staff finds the proposed Technical Specifications acceptable.

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ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

PRINCIPAL CONTRIBUTOR:

M. Slosson

Dated: October 14, 1987