

October 15, 1987

Docket No. 50-247

Mr. Murray Selman  
Vice President, Nuclear Power  
Consolidated Edison Company  
of New York, Inc.  
Broadway and Bleakley Avenue  
Buchanan, New York 10511

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Dear Mr. Selman:

The Commission has issued the enclosed Amendment No. 125 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 24, 1987 (TAC 65050).

The amendment revises the Technical Specifications to change the flow requirement for the Auxiliary Feedwater System in its limiting conditions for operation. The minimum required pumping capability of the auxiliary feedwater pumps is being revised from 400 gpm to 300 gpm.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager  
Project Directorate I-1  
Division of Reactor Projects, I/II

Enclosures:

- 1. Amendment No. 125 to DPR-26
- 2. Safety Evaluation

cc: w/enclosures  
See next page

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Mr. Murray Selman  
Consolidated Edison Company  
of New York, Inc.

Indian Point Nuclear Generating  
Station 1/2

cc:  
Mayor, Village of Buchanan  
236 Tate Avenue  
Buchanan, New York 10511

Mr. Jay Dunkleberger  
Office of Policy Analysis  
and Planning  
York State Energy Office  
Building 2, Empire State Plaza  
Albany, New York 12223

Jude Del Percio  
Manager of Regulatory Affairs  
Consolidated Edison Company  
of New York, Inc.  
Broadway and Bleakley Avenue  
Buchanan, New York 10511

Senior Resident Inspector  
U.S. Nuclear Regulatory Commission  
Post Office Box 38  
Buchanan, New York 10511

Brent L. Brandenburg  
Assistant General Counsel  
Consolidated Edison Company  
of New York, Inc.  
4 Irving Place - 1822  
New York, New York 10003

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Carl R. D'Alvia, Esquire  
Attorney for the Village of  
Buchanan, New York  
395 South Riverside Avenue  
Croton-on-Hudson, New York 10520

Director, Technical Development  
Programs  
State of New York Energy Office  
Agency Building 2  
Empire State Plaza  
Albany, New York 12223

Mr. Peter Kokolakis, Director  
Nuclear Licensing  
New York Power Authority  
123 Main Street  
White Plains, New York 10601

Mr. Walter Stein  
Secretary - NFSC  
Consolidated Edison Company  
of New York, Inc.  
4 Irving Place - 1822  
New York, New York 10003

Ezra I. Bialik  
Assistant Attorney General  
Environmental Protection Bureau  
New York State Department of Law  
2 World Trade Center  
New York, New York 10047



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 125  
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated March 24, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 125, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

*Robert A. Capra*

Robert A. Capra, Acting Director  
Project Directorate I-1  
Division of Reactor Projects, I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 15, 1987



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 125 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Page

3.4-1

Insert Page

3.4-1

### 3.4 STEAM AND POWER CONVERSION SYSTEM

#### Applicability

Applies to the operating status of the Steam and Power Conversion System.

#### Objective

To define conditions of the turbine cycle steam-relieving capacity. Auxiliary Feedwater System and City Water System operation is necessary to ensure the capability to remove decay heat from the core.

#### Specification

- A. The reactor shall not be heated above 350°F unless the following conditions are met:
- (1) A minimum ASME code approved steam relieving capability of twenty (20) main steam valves shall be operable (except for testing). With up to three (per steam generator) of the twenty main steam line code-approved safety relief valves inoperable, heat-up above 350°F and power operation is permissible provided either the inoperable valve(s) is restored to operable status or the Power Range Neutron Flux High Trip Setpoint is reduced per Table 3.4-1.
  - (2) Three auxiliary feedwater pumps, each capable of pumping a minimum of 300 gpm, must be operable.
  - (3) A minimum of 360,000 gallons of water in the condensate storage tank and a backup supply from the city water supply.
  - (4) Required system piping, valves, and instrumentation directly associated with the above components operable.
  - (5) The main steam stop valves are operable and capable of closing in five seconds or less.
  - (6) The total iodine activity of I-131 and I-133 on the secondary side of the steam generator shall be less than or equal to 0.15 uCi/cc.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 125 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

INTRODUCTION

As part of a reactor trip reduction program for Indian Point Unit 2, the Consolidated Edison Company of New York, Inc. (the licensee) submitted a proposal on March 27, 1987, to reduce the minimum flow that is required for each of three operable auxiliary feedwater pumps. This proposal is to change the required minimum flow in Item A(2) of Technical Specification (TS) 3.4 from 400 to 300 gpm.

DISCUSSION AND EVALUATION

Although this minimum flow specification applies to all three auxiliary feedwater pumps, it is only significant for the two electric motor driven pumps. The third pump, which is driven by a steam turbine rather than an electric motor, is designed to produce a flow of 800 gpm, and is expected to easily satisfy the currently required minimum flow of 400 gpm.

As determined in References 1 & 2, the minimum flow rate for the auxiliary feedwater pumps is set by the FSAR "Loss of Normal Feedwater Flow" transient. The present analysis of this transient, as described in FSAR Section 14.1.9, assumes an auxiliary feedwater flow rate of 300 gpm for the first 10 minutes and then 400 gpm thereafter. The licensee has redone this analysis and has submitted a revised Section 14.1.9 using a constant 300 gpm flow rate. The licensee states that "the analysis shows that with an auxiliary feedwater flow of 300 gpm, sufficient feedwater is available to dissipate decay heat without water relief from the primary system relief or safety valves and that the primary system variables never approach a departure from nucleate boiling condition".

The graphs included in the revised FSAR Section 14.1.9 show that the 300 gpm minimum flow rate is acceptable. Also, the analysis was performed using accepted methods. The method of analysis was designed to show that following a loss of normal feedwater, the auxiliary feedwater system is adequate to remove stored and residual heat to prevent water relief through the pressurizer relief valves. The method of analysis is described in the revised FSAR section. NRC acceptance criteria for the Loss of Normal Feedwater evaluation is contained in the Standard Review Plan Section 15.2. Thus, the staff finds that this revised analysis, and the associated TS change, is acceptable.

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The staff concludes that the required minimum auxiliary feedwater flow in Item A(2) of Technical Specification 3.4 can be reduced from 400 to 300 gpm without increasing the probability of consequences of an accident. Thus, the staff concludes that this change is acceptable.

#### ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

#### CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

#### REFERENCES

1. Letter from P. Zarakas, Consolidated Edison Company of New York, to S. Varga, USNRC, dated August 11, 1980.
2. Letter from S. Varga, USNRC, to J. O'Toole, Consolidated Edison Company of New York, dated August 30, 1982.

#### PRINCIPAL CONTRIBUTOR:

E. Lantz

Dated: October 15, 1987