

November 16, 1987

Docket No. 50-247

Mr. Murray Selman
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

DISTRIBUTION

Docket File
NRCPDR
Local PDR
PDI-1 Rdg.
OGC
SVarga
BBoger
CVogan
GPA/PA
PSwetland

MSlosson
EJordan
JPartlow
EButcher
TBarnhart(4)
DHagan
Wanda Jones
ACRS(10)
ARM/LFMB
JWermiel

Dear Mr. Selman:

The Commission has issued the enclosed Amendment No.127 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated July 17, 1987 (TAC 65865).

The amendment revises the Technical Specifications to allow a containment integrated leak rate test duration of less than 24 hours as described in Bechtel Power Corporation Topical Report, BN-TOP-1, Revision 1.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No.127 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

PDI-1
CVogan *CV*
9/24/87

PDI-1 *MS*
MSlosson
9/28/87

JW
SPLB
JWermiel
9/29/87

MS
OGC
9/16/87

RAC
PDI-1
RCapra
9/16/87
H

8711200069 871116
PDR ADDCK 05000247
P PDR

Mr. Murray Selman
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

cc:

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, New York 10511

Mr. Jay Dunkleberger
Office of Policy Analysis
and Planning
York State Energy Office
Building 2, Empire State Plaza
Albany, New York 12223

Jude Del Percio
Manager of Regulatory Affairs
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 38
Buchanan, New York 10511

Brent L. Brandenburg
Assistant General Counsel
Consolidated Edison Company
of New York, Inc.
4 Irving Place - 1822
New York, New York 10003

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Carl R. D'Alvia, Esquire
Attorney for the Village of
Buchanan, New York
395 South Riverside Avenue
Croton-on-Hudson, New York 10520

Director, Technical Development
Programs
State of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Mr. Peter Kokolakis, Director
Nuclear Licensing
New York Power Authority
123 Main Street
White Plains, New York 10601

Mr. Walter Stein
Secretary - NFSC
Consolidated Edison Company
of New York, Inc.
4 Irving Place - 1822
New York, New York 10003

Ezra I. Bialik
Assistant Attorney General
Environmental Protection Bureau
New York State Department of Law
2 World Trade Center
New York, New York 10047



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated July 17, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

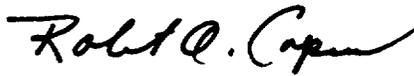
8711200071 871116
PDR ADOCK 05000247
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 127, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Acting Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 16, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 127 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

4.4-1

4.4-7

Insert Pages

4.4-1

4.4-7

4.4 CONTAINMENT TESTS

Applicability

Applies to containment leakage.

Objective

To verify that potential leakage from the containment is maintained within acceptable values.

Specification

A. Integrated Leakage Rate

1. Test

- a. A full pressure integrated leakage rate test shall be performed at intervals specified in A.3 at the peak accident pressure (P_a) of 47 psig minimum.
- b. A test duration of 24 hours, or an NRC approved reduced duration methodology, as described in BN-TOP-1, Revision 1, shall be used. The test shall be extended a sufficient period of time to verify, by superimposing a known leak rate on the containment, the validity and accuracy of the leakage rate results.
- c. A general inspection of the accessible interior and exterior surfaces of the containment structures and components shall be performed prior to performing an integrated leak test to uncover any evidence of structural deterioration which may affect either the containment structural integrity or leak tightness. If there is evidence of structural deterioration, integrated leakage rate tests shall not be performed until corrective action is taken. Such structural deterioration and corrective actions taken shall be reported as part of the test report.

continuously pressurizing both the penetrations and the channels over all containment liner welds. These channels were independently leak-tested during construction.

The safety analysis has been performed on the basis of a leakage rate of 0.10 w/o per day for 24 hours. With this leakage rate and with minimum containment engineered safeguards operating, the public exposure would be well below 10CFR100 values in the event of the design basis accident.⁽³⁾

The performance of a periodic integrated leakage rate test during plant life provides a current assessment of potential leakage from the containment. In order to provide a realistic appraisal of the integrity of the containment under accident conditions, the containment isolation valves are to be closed in the normal manner and without preliminary exercising or adjustments.

The minimum duration of 24 hours for the integrated leakage rate test is established to attain the desired level of accuracy and to allow for daily cyclic variation in temperature and thermal radiation. If an ILRT of a duration less than 24 hours is attempted, the criteria of the Bechtel Topical Report, BN-TOP-1, Revision 1, will be met.

The frequency of the periodic integrated leakage rate test is keyed to the schedule for major shutdowns for inservice inspection and refueling. The specified frequency of periodic integrated leakage rate testing is based on the following major considerations.

First is the low probability of leaks in the liner, because of

- (a) the tests of the leak-tight integrity of the welds during erection;
- (b) conformance of the complete containment to a low leakage rate limit at 47 psig or higher during pre-operational testing; and
- (c) absence of any significant stresses in the liner during reactor operation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 127 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

INTRODUCTION

By letter dated July 17, 1987, Consolidated Edison Company of New York, Inc. (the licensee) requested a change to plant Technical Specification (TS) Section 4.4 regarding the Containment Integrated Leakage Rate Test (ILRT) for Indian Point Nuclear Generating Unit No. 2. The proposed change will allow an ILRT duration as specified in TS Section 4.4 and its associated Basis of less than 24 hours as described in Bechtel Power Corporation Topical Report, BN-TOP-1, Revision 1.

DISCUSSION AND EVALUATION

TS Section 4.4 currently specifies that the ILRT duration be at least 24 hours. The proposed change would permit reducing the test duration on the basis of compliance with a methodology which has been approved by the NRC staff. By letter dated February 1, 1973, the staff found BN-TOP-1, Revision 1, acceptable even though it provides for shorter duration testing criteria for ILRTs. Based on previous staff approval of BN-TOP-1, the staff concludes that the proposed TS change to permit ILRT in accordance with Bechtel Topical Report BN-TOP-1, Revision 1, is acceptable.

The licensee requested that the Technical Specifications be revised to accept other NRC approved methodologies. Currently, there are no other approved methodologies applicable to Indian Point 2. As a result, the proposed Technical Specifications have been revised to delete the words concerning other approved methodologies. This change was discussed with the licensee and found to be acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to surveillance of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents

8711200074 B71116
PDR ADDCK 05000247
P PDR

that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

PRINCIPAL CONTRIBUTOR:

M. Slosson

Dated: November 16, 1987

November 16, 1987

MEMORANDUM FOR: Sholly Coordinator DISTRIBUTION
 FROM: Robert A. Capra, Acting Director Docket File
 Project Directorate I-1 PDI-1 Rdg.
 Division of Reactor Projects, I/II CVogan
MSLosson
JScinto
 SUBJECT: REQUEST FOR PUBLICATION IN BI-WEEKLY FR NOTICE - NOTICE
 OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Consolidated Edison Company of New York, Docket No. 50-247, Indian Point
 Nuclear Generating Unit No. 2, Westchester County, New York

Date of application for amendment: July 17, 1987

Brief description of amendment: The amendment revises the Technical
 Specifications to allow a containment integrated leak rate test duration of
 less than 24 hours as described in Bechtel Power Corporation Topical Report,
 BN-TOP-1, Revision 1.

Date of issuance: November 16, 1987

Effective date: November 16, 1987

Amendment No.: 127

Facility Operating License No. DPR-26: Amendment revised the Technical
 Specifications.

Date of initial notice in Federal Register:

The Commission's related evaluation of the amendment is contained in a
 Safety Evaluation dated November 16, 1987.

No significant hazards consideration comments received: No

Local Public Document Room location: White Plains Public Library, 100
 Martine Avenue, White Plains, New York, 10610.

8711190372 871116
 CF ADDCK 05000247
 CF

Robert A. Capra, Acting Director
 Project Directorate I-1
 Division of Reactor Projects, I/II

PDI-1
 CVogan
 9/24/87

PDI-1 ms
 MSlOSSon
 9/28/87

[Handwritten signature]
 10/9/87

[Handwritten initials]
 PDI-1
 RCapra
 8/16/87