

November 18, 1987

Docket No. 50-247

Mr. Murray Selman
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

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Dear Mr. Selman:

The Commission has issued the enclosed Amendment No. 129 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated May 5, 1977, as supplemented February 16, 1984 and superceded July 18, 1986, as revised July 14, 1987 (TAC 51913).

The amendment revises the Technical Specifications to incorporate a commitment to an Inservice Inspection and Testing Program and periodic updating consistent with 10 CFR 50.55a. The issuance of this amendment also eliminates the need for issuance of your amendment applied for by letter dated August 29, 1983. This application requested an amendment concerning reactor coolant pump casing and welds. With issuance of this amendment, the August 29, 1983 amendment is no longer applicable.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

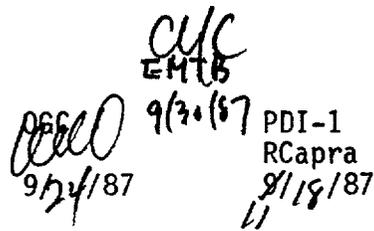
Enclosures:

1. Amendment No. 129 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

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of New York, Inc.

Indian Point Nuclear Generating
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 129
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated May 5, 1977, as supplemented February 16, 1984 and superceded July 18, 1986, as revised July 14, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 129, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Acting Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 18, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 129 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

(4.2-1 thru 28)

Insert Pages

4.2-1

4.2-2

Table 4.2-1

Table 4.2-1 (continued)

4.2 Inservice Inspection and Testing

Applicability

Applies to the inservice inspection of Quality Groups* A, B, and C components and the inservice testing of pumps and valves whose function is required for safety.

Objective

To provide assurance of the continued integrity and/or operability of those structures, systems, and components to which this specification is applicable.

Specification

4.2.1 Inservice Testing

Inservice testing of pumps and valves whose function is required for safety shall be performed in accordance with the applicable edition and addenda of Section XI of the ASME Boiler and Pressure Vessel Code as required by 10CFR50, Section 50.55a(g), except where specific written relief pursuant to 10CFR50, Section 50.55a(g)(6)(i) has been granted.

4.2.2 Inservice Inspection

Inservice inspection of Quality Group* A, B, and C components shall be performed in accordance with the applicable edition and addenda of Section XI of the ASME Boiler and Pressure Vessel Code as required by 10CFR50, Section 50.55a(g), except where specific written relief pursuant to 10CFR50, Section 50.55a(g)(6)(i) has been granted.

4.2.3 Primary Pump Flywheels

The flywheels shall be visually examined at the first refueling. At each subsequent refueling, one different flywheel shall be examined by ultrasonic methods. The examinations schedules are shown in Table 4.2-1.

4.2.4 Reactor Vessel Special Inspection

1. Interval of Inspection:

The reactor vessel shall be examined during the second ten year interval in the area of the vessel weld located approximately 236 inches below the reactor vessel flange at 345° azimuth. This area shall be re-examined during the three successive inspection periods as defined in accordance with IWB-2410 of the 1980 ASME Boiler & Pressure Vessel Code, Section XI, as modified below.

The examination schedule may revert to the original inspection schedule per IWB-2410 if:

- (i) The additional examinations reveal that the indications remain essentially unchanged over 3 successive inspections, or
- (ii) Any additional examination utilizing-ultrasonic techniques per IWA-2232, or alternative techniques per IWA-2240, as supplemented by prior examination, demonstrate that the reflector meets the acceptance standards of IWB-3510. Such demonstration shall be submitted for NRC review and approval. Upon receipt of NRC concurrence, this special inspection requirement (4.2.4 in its entirety) shall become void.

2. Reporting Requirements:

The reactor vessel inservice inspection program shall be forwarded to NRC 180 days prior to plant shutdown during which the inspection is scheduled to be accomplished. Inspection results shall be forwarded for NRC review and approval 15 days prior to plant startup.

*Quality Group classification is in accordance with Revision 3 of Regulatory Guide 1.26.

References

- (1) Letter from Robert W. Reid of NRC to William J. Cahill of Consolidated Edison dated April 22, 1976
- (2) Letter from Robert W. Reid of NRC to William J. Cahill of Consolidated Edison dated November 17, 1976
- (3) Letter from William J. Cahill of Consolidated Edison to Robert W. Reid of NRC dated May 27, 1976

Table 4.2-1

<u>No. Item</u>	<u>Examination Category</u>	<u>Components and Part to be Examined</u>	<u>Method</u>	<u>Extent of Examination (Percent in 10 Year Interval)</u>	<u>Remarks</u>
4.2.3	N/A	Primary pump flywheel	V & UT	See Remarks	The flywheels shall be visually examined at the first refueling. At each subsequent refueling, one different flywheel shall be examined by ultrasonic methods.
4.2.4	N/A	Reactor Vessel Special Inspection Area	UT	See Remarks	<p>The reactor vessel shall be examined during the second ten year interval in the area of the vessel weld located approximately 236 inches below the reactor vessel flange at 345° azimuth. This area shall be reexamined during the three successive inspection periods as defined in accordance with IWB-2410 of the 1980 ASME Boiler & Pressure Vessel Code, Section XI as modified below.</p> <p>The examination schedule may revert to the original inspection schedule per IWB-2410 if:</p> <p>(1) The additional examinations reveal that the indications remain unchanged over 3 successive inspections, or</p>

Table 4.2-1 (cont'd)

<u>No. Item</u>	<u>Examination Category</u>	<u>Components and Part to be Examined</u>	<u>Method</u>	<u>Extent of Examination (Percent in 10 Year Interval)</u>	<u>Remarks</u>
					<p>(ii) Any additional examination utilizing ultrasonic techniques per IWA-2232, or alternative techniques per IWA-2240, as supplemented by prior examination, demonstrate that the reflector meets the acceptance standards of IWB-3510. Such demonstration shall be submitted for NRC review and approval. Upon receipt of NRC concurrence, this special inspection requirement shall become void.</p>



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 129 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

INTRODUCTION

By letters dated May 5, 1977, as supplemented February 16, 1984, and superseded July 18, 1986, as revised July 14, 1987, Consolidated Edison Company of New York, Inc. submitted an application to amend the Indian Point 2 Technical Specifications to incorporate a commitment to an Inservice Inspection and Testing Program and periodic updating, consistent with 10 CFR 50.55a.

DISCUSSION AND EVALUATION

By letters dated April 22 and November 17, 1976, the NRC staff provided Consolidated Edison with information concerning the fact that the Inservice Inspection and Testing (ISI and IST) requirements for ASME Code Class 1, 2, and 3 components for nuclear power plants delineated in 10 CFR 50.55a were changed by a revision to the regulations published on February 27, 1976. The revised regulations require ISI and IST to be performed in accordance with the examination and testing requirements set forth in Section XI of the ASME Boiler and Pressure Vessel Code and Addenda.

In addition, the letters requested that Consolidated Edison request an amendment to the Indian Point 2 Technical Specifications to be consistent with the change in regulations.

By letter dated July 18, 1986 as revised July 14, 1987, Consolidated Edison submitted Technical Specifications in conformance with the 1976 letters. The proposed specifications indicated that ISI and IST of Quality Group A, B and C components shall be performed in accordance with the applicable edition and addenda of Section XI as required by 10 CFR 50.55a(g) except where specific written relief pursuant to CFR 50.55a(g)(6)(i) has been granted. The Quality Group classification is in accordance with Revision 3 of Regulatory Guide 1.26. The Quality Groups A, B, and C correspond to ASME Code Class 1, 2, and 3, respectively.

The staff finds the proposed Technical Specifications to be acceptable.

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In addition, issuance of the proposed amendment eliminates the need for issuance of Consolidated Edison's August 29, 1983 amendment application concerning reactor coolant pump casing and welds. With issuance of the above proposed Technical Specifications, the August 29, 1983 application is no longer applicable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the surveillance of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

PRINCIPAL CONTRIBUTOR:

M. Slosson
T. Sullivan

Dated: November 18, 1987