

May 28, 1985

Docket No. 50-286

Mr. John C. Brons
Senior Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

<u>Distribution</u>	
<u>Docket file</u>	NRC PDR
ORB#1 RDG	L PDR
Gray file (4)	HThompson
CParrish	KJohnston
DNeighbors	OELD
LHarmon	EJordan
BGrimes	JPartlow
TBarnhart (4)	WJones
EButcher	ACRS (10)
OPA, CMiles	RDiggs
SMiner	

Dear Mr. Brons:

The Commission has issued the enclosed Amendment No.56 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated December 3, 1984.

The amendment would revise Section 3.7 of the Technical Specifications to define the Limiting Conditions for Operation of systems, subsystems, trains, components and devices supplied by an inoperable normal or emergency power source.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/JDNeighbors

Joseph D. Neighbors, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 56 to DPR-64
2. Safety Evaluation

cc: w/enclosures
See next page

*SEE PREVIOUS WHITE FOR CONCURRENCE

ORB#1:DL*
CParrish
05/06/85

ORB#1:DL
KJohnston/ts
05/07/85

ORB#1:DL
DNeighbors
05/07/85

ORB#4:DL
SMiner
05/07/85

BC-ORB#1:DL
SVarga
05/08/85

OELD
05/11/85

AD:OR:DL
Glatnas
05/15/85

*M.K.
by J.C. Brons
partly phone
on 5/28/85
(see concurrence
on monthly memo)*

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Mr. John C. Brons
Senior Vice President - Nuclear Generation
Power Authority of the State of New York
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Dear Mr. Brons:

The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated December 3, 1984.

The amendment would revise Section 3.7 of the Technical Specifications to define the Limiting Conditions for Operation of systems, subsystems, trains, components and devices supplied by an inoperable normal or emergency power source, as provided by the Standard Technical Specifications.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

Joseph D. Neighbors, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

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cc: w/enclosures
See next page

ORB#1:DL CParrish 5/6/85	ORB#1:DL KJohnston/ts 04/ /85	ORB#1:DL DNeighbors 04/ /85	ORB#4:DL SMiner 04/ /85	BC-ORB#1:DL SVarga 04/ /85
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 28, 1985

Docket No. 50-286

Mr. John C. Brons
Senior Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

Dear Mr. Brons:

The Commission has issued the enclosed Amendment No.56 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated December 3, 1984.

The amendment would revise Section 3.7 of the Technical Specifications to define the Limiting Conditions for Operation of systems, subsystems, trains, components and devices supplied by an inoperable normal or emergency power source.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Joseph D. Neighbors".

Joseph D. Neighbors, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 56 to DPR-64
2. Safety Evaluation

cc: w/enclosures
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated December 3, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 56, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 28, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 56

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

3.7-3a
3.7-5
3.7-6

Insert Pages

3.7-3a
3.7-5
3.7-6

4. Two operable diesel generators together with total underground storage containing a minimum of 5676 gallons of fuel.

G. When a system, subsystem, train, component or device is determined to be inoperable solely because its emergency power source is inoperable, or solely because its normal power source is inoperable, it may be considered operable for the purpose of satisfying the requirements of its applicable specification provided: (1) its corresponding normal or emergency power source is operable; and (2) all of its redundant system(s), subsystem(s), train(s), components(s) and device(s) are operable or likewise satisfy the requirements of the specification.

Basis

The electrical system equipment is arranged so that no single contingency can inactivate enough safeguards equipment to jeopardize the plant safety. The 480-volt equipment is arranged on 4 buses. The 6900-volt equipment is supplied from 6 buses.

The Buchanan Substation has both 345 KV and 138 KV transmission circuits which are capable of supplying startup, normal operation, shutdown and/or engineered safeguards loads.

The 138 KV supplies or the gas turbines are capable of providing sufficient power for plant startup. Power via the station auxiliary transformer can supply all the required plant auxiliaries during normal operation, if required.

In addition to the unit transformer, four separate sources supply station service power to the plant. (1)

Any two of three diesel generators, the station auxiliary transformer or the separate 13.8 to 6.9 KV transformer are each capable of supplying the minimum safeguards loads, and therefore provide separate sources of power immediately available for operation of these loads. Thus the power supply system meets the single failure criteria required of safety systems. To provide maximum assurance that the redundant or alternate power supplies will operate if required to do so, the redundant or alternate power supplies are verified operable prior to initiating repair of the inoperable power supply. Continued plant operation is governed by the specified allowable time period for the power source, not the specified allowable time period for those items determined to be inoperable solely because of the inoperability of its normal or emergency power source provided the conditions defined in specification 3.7.G are satisfied. These conditions assure that the minimum required safeguards will be operable. If it develops that (a) the inoperable power supply is not repaired within the specified allowable time period, or (b) a second power supply in the same or related category is found to be inoperable, the reactor, if critical, will initially be brought to the hot shutdown condition utilizing normal operating procedures to provide for reduction of the decay heat from the fuel, and consequent reduction of cooling requirements after a postulated loss-of-coolant accident. If the reactor was already subcritical, the reactor coolant system temperature and pressure will be maintained within the stated values in order to limit the amount of stored energy in the Reactor Coolant System. The stated tolerances provide a band for operator control. After a limited time in hot shutdown, if the malfunction(s) are not corrected, the reactor will be brought to the cold shutdown condition, utilizing normal shutdown and cool-down procedures. In the cold shutdown condition there is no possibility of an accident that would release fission products or damage the fuel elements.

Conditions of a system-wide blackout could result in a unit trip. Since normal off-site power supplies as required in Specification 3.7.A.1 are not available for startup, it is necessary to be able to black start the unit with gas turbines providing the incoming power supplies as a first step in restoring the system to an operable status and restoring power to customers for essential services. Specification 3.7.C provides for startup using 37 MW's of gas turbine power (nameplate rating at 80°F) which is sufficient to carry out a normal plant startup. A system-wide blackout is deemed to exist when the majority of Con Edison electric generating facilities are shutdown due to an electrical disturbance and the remainder are incapable of supplying the system therefore necessitating major load shedding.

Since the backup lighting supply is stripped on safety injection, the requirement that not more than one 120 volt A.C. instrument bus be energized from the backup lighting supply is to assure minimum operable containment spray actuation channels.

As a result of an investigation of the effect components that might become submerged following a LOCA may have on ECCS, containment isolation and other safety-related functions, a fuse and a locked open circuit breaker were provided on the electrical feeder to emergency lighting panel 318 inside containment. With the circuit breaker in the open position, containment electrical penetration H-70 is de-energized during the accident condition. Personnel access to containment may be required during power operation. Since it is highly improbable that a LOCA would occur during this short period of time, the circuit breaker may be closed during that time to provide emergency lighting inside containment for personnel safety.

References

- 1) FSAR - Section 8.2.1
- 2) FSAR - Section 8.2.3



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

Introduction

An NRC letter to all power reactor licensees, dated April 10, 1980, presented the staff's concern about a possible misunderstanding regarding the use of the term "OPERABLE" as it applies to the single failure criterion for safety systems in power reactors. The purpose of the NRC letter was to clarify the meaning of the term and to request that the licensee take specific actions to assure that it is appropriately applied. By License Amendment No. 32, dated September 5, 1980, we granted the licensee's amendment, requested by letter dated May 23, 1980, implementing the Standard Technical Specifications definition of OPERABLE.

During a recent review it became clear that Amendment No. 32 did not fully satisfy the intent of the April 10, 1980 generic letter. While it provided an adequate definition of OPERABLE, Amendment No. 32 did not adequately address the concerns of Standard Technical Specifications 3.0.3 and 3.0.5. By letter dated October 22, 1984, the staff requested that the licensee submit a license amendment to resolve the remaining concerns. The licensee submitted an amendment request dated December 3, 1984, addressing these concerns.

Discussion and Evaluation

We have completed our review of the December 3, 1984 submittal. In essence, the April 10, 1980 generic letter covers three areas. These are: (1) Paragraph 1.0, Definition of Operability, (2) Paragraph 3.0.3, Limiting Condition for Operation for circumstances which are in excess of those addressed in the Technical Specifications, and (3) Paragraph 3.0.5, Limiting Conditions for Operation for inoperable normal or emergency power sources. License Amendment No. 32 revised the IP-3 Technical Specifications to reflect an acceptable Definition of Operability.

The licensee's submittals of May 23, 1980 and December 3, 1984 both state that the intent of Standard Technical Specifications paragraph 3.0.3 is satisfied by the first paragraph of Section 3 of the Indian Point 3 Technical Specifications. The intent of Paragraph 3.0.3 is to provide the action to be taken for circumstances not directly provided for in the ACTION statements and whose occurrence would violate the intent of the specification, i.e., to ensure that action is provided in situations where there is a loss of both trains of a redundant system.

The first paragraph of Section 3 of the IP-3 Technical Specifications states,

"For the cases where no exception time is specified for inoperable components, this time is assumed to be zero".

The "exception time" is interpreted in plant operations procedures to be the equipment out-of-service time specified for a component by a Limiting Condition of Operation. In its December 3, 1984 submittal, the licensee provided an example of a circumstance necessitating the invocation of Technical Specification Section 3 as follows:

"Technical Specification 3.3.A.3 provides limitations on the Safety Injection and Residual Heat Removal Systems, which stipulate, in part, that the reactor shall not exceed 350°F unless, three safety injection pumps together with their associated piping and valves are operable. Technical Specification 3.3.A.4 states that Technical Specification 3.3.A.3 may be modified to allow one safety injection pump to be out of service provided the pump is restored to an operable status within 24 hours and the remaining two pumps are demonstrated to be operable. Technical Specification 3.3.A.5 states that if the Safety Injection and Residual Heat Removal Systems are not restored to meet the requirements of 3.3.A.3 within the time periods specified in 3.3.A.4, then if the reactor is critical, it shall be in the hot shutdown condition within the four hours and the cold shutdown condition within the following 24 hours. However if the requirements of 3.3.A.4 are exceeded, that is, a second inoperable safety injection pump, then the out-of-service time is assumed to be zero and the plant is immediately brought to the cold shutdown condition in accordance with the applicable prompt shutdown requirements, per Technical Specifications Section 3."

In view of this evidence it is apparent that the first paragraph of Section 3 of the IP-3 Technical Specifications satisfies the intent of Standard Technical Specifications paragraph 3.0.3, and is, therefore, acceptable.

In their December 3, 1984 submittal, the licensee has requested the addition of paragraph 3.7.G. This paragraph differs from Standard Technical Specifications 3.0.5 in that it does not provide an ACTION statement for the situation where conditions (1) and (2) are not satisfied, i.e., the unit placed in at least Hot Standby within 1 hour, in at least Hot Shutdown within the next 6 hours, and in at least Cold Shutdown within the following 30 hours. However, in a situation where both conditions (1) and (2) are not satisfied, the first paragraph of Section 3 applies and the plant is brought to the cold shutdown condition in accordance with the shutdown requirements of the most limiting action statement of the applicable systems, subsystems, trains, components or devices.

It was not the staff's intention that the Limiting Conditions for Operation at IP-3 be changed to conform verbatim with the Standard Technical Specifications. The licensee's submittal meets the intent of the

April 10, 1980 letter since the proposed change is in accordance with the NRC guidance provided with respect to the definition of OPERABLE and is, therefore, acceptable.

Summary

The intent of paragraph 1.6 of the Standard Technical Specifications is satisfied by the definition of OPERABLE issued by Amendment No. 32. The intent of paragraph 3.0.3 is satisfied by the first paragraph of Section 3 of the IP-3 Technical Specifications. The intent of paragraph 3.0.5 is satisfied by the changes to the IP-3 Technical Specifications enclosed in this amendment. Therefore, the concerns presented in the generic letter of April 10, 1980 have been addressed and are satisfied.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 28, 1985

Principal Contributor:

Ken Johnston