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August 1, 1988

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to DPR-26

Docket Nos. 50-003
and 50-247

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Mr. Stephen B. Bram
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

Dear Mr. Bram:

The Commission has issued the enclosed Amendment No.40 to Facility Operating License No. DPR-5 and Amendment No.134 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit Nos. 1 and 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated June 15, 1988 (TAC 68645).

The amendments revise the Indian Point 1 and 2 Technical Specifications to delete Figures 3.1 and 3.2 from Unit 1 Technical Specifications and Figures 6.2.1 and 6.2.2 from Unit 2 Technical Specifications. The removal was submitted in accordance with Generic Letter 88-06, "Removal of Organization Charts from Technical Specification Administrative Control Requirements."

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

~~Signature~~

David Langford, Acting Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No. 40 to DPR-5
2. Amendment No. 134 to DPR-26
3. Safety Evaluation

cc: w/enclosures
See next page

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Mr. Stephen B. Bram
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-003

INDIAN POINT NUCLEAR GENERATING UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 40
License No. DPR-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated June 15, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-5 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 40, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 1, 1988



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. DPR-5

DOCKET NO. 50-003

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3	3
Figure 3.1	4
Figure 3.2	-

3.0 Administrative and Procedural Safeguards

3.1 Organization

- 3.1.1 Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.
- a. Lines of authority, responsibility, and communication shall established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Updated FSAR.
 - b. The General Manager-Nuclear Power Generation shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant. He shall delegate in writing the succession to this responsibility during his absence.
 - c. The Vice President-Nuclear Power shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety. He shall delegate in writing the succession to this responsibility during his absence.
 - d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.
 - e. The operation of the facility, the operating organization, the procedures for operation, and modifications to the facility shall be subject to review by the Station Nuclear Safety Committee. The committee shall report to the Vice President, Nuclear Power.
 - f. The Nuclear Facilities Safety Committee shall function to provide independent review and audit of designated activities in areas of nuclear engineering, chemistry, radiochemistry, metallurgy and non-destructive testing, instrumentation and

control, radiological safety, mechanical and electrical engineering, administrative controls and quality assurance practices, and radiological environmental effects.

- g. All fuel handling shall be under the direct supervision of a licensed operator.*
- h. The Support Facilities Supervisor is responsible for operations at the Unit No. 1 facility.
- i. The Operations Manager shall hold a senior reactor operator license.

3.2 Operating Instructions and Procedures

- 3.2.1 No fuel will be loaded into the reactor core or moved into the reactor containment building without prior review and authorization by the Nuclear Regulatory Commission.
- 3.2.2 Detailed written instruction setting forth procedures used in connection with the operation and maintenance of the nuclear power plant shall conform to the Technical Specifications.
- 3.2.3 Operation and maintenance of equipment related to safety when there is no fuel in the reactor shall be in accordance with written instructions.

* Licensed operator for IP-1 or IP-2



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 134
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated June 15, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 134, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 1, 1988



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 134 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
vii	vii
6-1	6-1
6-1(a)	6-2
Figure 6.2.1	6-3
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6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Vice President-Nuclear Power shall be responsible for overall facility activities and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The General Manager-Nuclear Power Generation shall be responsible for facility operations and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

6.2.1 Facility Management and Technical Support

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Updated FSAR.
- b. The General Manager-Nuclear Power Generation shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President-Nuclear Power shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 Facility Staff

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor startup, scheduled reactor shutdown, and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be onsite when fuel is in the reactor.
- e. ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling. This individual shall have no other concurrent responsibilities during this operation.

- f. A Fire Brigade of at least five members shall be maintained on-site at all times.* This excludes four members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency. During periods of cold shutdown, the Fire Brigade will exclude two members of the minimum shift crew.
- g. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions (e.g., licensed Senior Operators, licensed Operators, health physicists, auxiliary operators, and key maintenance personnel).

The amount of overtime worked by unit staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).

- h. The Operations Manager shall hold a senior reactor operator license.

* Fire Brigade composition may be one member less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. DPR-5
AND RELATED TO AMENDMENT NO. 134 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NOS. 1 AND 2
DOCKET NOS. 50-003 AND 50-247

INTRODUCTION

By letter dated June 15, 1988, the Consolidated Edison Company of New York, Inc. (the licensee) proposed changes to Technical Specification (TS) 3.1, Organization for Indian Point Unit No. 1 and Technical Specification 6.2, Organization for Indian Point Unit No. 2. The proposed changes would remove Figures 3.1 and 3.2 from the Unit 1 Technical Specification and Figures 6.2.1 and 6.2.2 from the Unit 2 Technical Specifications and replace them with a narrative description of the offsite and onsite organization functional requirements in TS 3.1 for Unit 1 and TS 6.2 for Unit 2. Guidance for these proposed changes to the TS was provided to licensees and applicants by Generic Letter 88-06, dated March 22, 1988.

BACKGROUND

Consistent with the guidance provided in the Standard Technical Specifications, Specifications 3.1 for Unit 1 and 6.2.1 and 6.2.2 of the administrative control requirements for Unit 2 have referenced offsite and unit (onsite) organization charts that are provided as figures to these sections. On a plant specific basis, these organization charts have been provided by applicants and included in the TS issued with the operating license. Subsequent restructuring or either the offsite or unit organizations, following the issuance of an operating license, has required licensees to submit a license amendment for NRC approval to reflect the desired changes in these organizations. As a consequence, organizational changes have necessitated the need to request an amendment of the operating license.

Because of these limitations on organizational structure, the nuclear industry has highlighted this an area for improvement in the TS. The Shearon Harris licensee proposed changes to remove organization charts from its TS under the lead-plant concept that included the endorsement of the proposed changes by the Westinghouse Owners Group. In its review of the Shearon Harris proposal, the staff concluded that most of the essential elements of offsite and onsite

organization charts are captured by other regulatory requirements, notably, Appendix B to 10 CFR 50. However, there were aspects of the organization structure that are important to ensure that the administrative control requirements of 10 CFR 50.36 would be met and that would not be retained with the removal of the organization charts. The applicable regulatory requirements are those administrative controls that are necessary to ensure safe operation of the facility. Therefore, those aspects of organization charts for Shearon Harris that were essential for conformance with regulatory requirements were added (1) to Specifications 6.2.1 to define functional requirements for the offsite and onsite organizations and (2) to Specification 6.2.2 to define qualification requirements of the unit staff.

By letter dated January 27, 1988, the staff issued Amendment No. 3 to Facility Operating License NFP-63 for the Shearon Harris Nuclear Power Plant that incorporated these changes to their TS. Subsequently the staff developed guidance on an acceptable format for license amendment requests to remove the organization charts from TS. Generic Letter 88-06 provided this guidance to all power reactors.

EVALUATION

The licensee's proposed changes to its TS are in accordance with the guidance provided by Generic Letter 88-06 and addressed the items listed below.

1. Specification 3.1 for Unit 1 was revised to delete references to Figures 3.1 and 3.2 that were removed from the TS. Specifications 6.2.1 and 6.2.2 for Unit 2 were revised to delete the references to Figures 6.2-1 and 6.2-2 that were removed from the TS.
2. Functional requirements of the offsite and onsite organizations were defined and added to Specification 3.1 for Unit 1 and 6.2.1 for Unit 2, and they are consistent with the guidance provided in Generic Letter 88-06. The TS notes that implementation of these requirements is documented in the Updated FSAR.
3. The senior reactor operator license qualified position identified on the organization chart for the unit staff, was noted as being required by Specification 3.1.1 for Unit 1 and 6.2.2 for Unit 2. Therefore, this requirement will be retained as a requirement following the removal of the organization chart for the unit staff.
4. Consistent with requirements to document the offsite and onsite organization relationships in the form of organization charts, the licensee has confirmed that this documentation has been designated for inclusion to the next update of the Updated FSAR.
5. The licensee has confirmed that no specifications, other than those noted in item 1. above, include references to the figures of the organization charts that are being removed from TS for their plant. Hence, this is not an applicable consideration, with regard to the need to redefine referenced requirements as a result of the removal of these figures.

On the basis of this review of the above items, the staff concludes that the licensee has provided an acceptable response to these items as addressed in the NRC guidance on removing organization charts from the administrative control requirements of the TS. Furthermore, the staff finds that these changes are consistent with the staff's generic finding on the acceptability of such changes as noted in Generic Letter 88-06. Accordingly, the staff finds the proposed changes to be acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, or administrative procedures or requirements. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

The Commission made proposed determinations that the amendments involve no significant hazards consideration, which were published in the Federal Register (53 FR 24533) on June 29, 1988. The Commission consulted with the State of New York. No public comments were received, and the State of New York did not have any comments.

On the basis of the considerations discussed above the staff concludes that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

PRINCIPAL CONTRIBUTORS:

T. Dunning, NRR
M. Slosson, NRR
D. Langford, NRR

Dated: August 1, 1988