

Docket No. 50-247

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Mr. William J. Cahill, Jr.
 Vice President
 Consolidated Edison Company
 of New York, Inc.
 4 Irving Place
 New York, New York 10003

Dear Mr. Cahill:

The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. This amendment consists of changes to the Technical Specifications in response to application transmitted by letter dated June 4, 1979.

The amendment revises the Technical Specifications to permit waiving the turbine stop and control vavle testing at end of cycle.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A. Schwencer, Chief
 Operating Reactors Branch #1
 Division of Operating Reactors

- Enclosures:
 1. Amendment No. to DPR-26
 2. Safety Evaluation
 3. Notice of Issuance

*Cahill
 CCF*

cc: w/enclosures
 See next page

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	SURNAME	LNOlshan:jb	CSParrish	PGlock	R.Vollmer	S.H.Lewis	ASchwencer
	DATE	06/6/79	06/6/79	06/6/79	06/7/79	06/7/79	06/6/79



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Docket No. 50-247

Mr. William J. Cahill, Jr.
Vice President
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

June 7, 1979

Dear Mr. Cahill:

The Commission has issued the enclosed Amendment No. 54 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. This amendment consists of changes to the Technical Specifications in response to application transmitted by letter dated June 4, 1979.

The amendment revises the Technical Specifications to permit waiving the turbine stop and control valve testing at end of cycle.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

1. Amendment No. 54 to DPR-26
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

Mr. William J. Cahill, Jr.
Consolidated Edison Company of New York, Inc.

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June 7, 1979

cc: White Plains Public Library
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Region II Office
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New York, New York 10007



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated June 4, 1979 complies with the standards and requirements of the Atomic Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 54, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance:
June 7, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 54

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

Insert Pages

Table 4.1-3

Table 4.1-3

TABLE 4.1-3

FREQUENCIES FOR EQUIPMENT TESTS

	<u>Check</u>	<u>Frequency</u>	<u>Maximum Time Between Tests</u>
1. Control Rods	Rod drop times of all control rods	Each refueling shutdown	**
2. Control Rods	Partial movement of all control rods	Every 2 weeks during reactor critical operations	20 days
3. Pressurizer Safety Valves	Set point	Each refueling shutdown	**
4. Main Steam Safety Valves	Set point	Each refueling shutdown	**
5. Containment Isolation System	Automatic Actuation	Each refueling shutdown	**
6. Refueling System Interlocks	Functioning	Each refueling shutdown prior to refueling operation	NA*
7. Primary System Leakage	Evaluate	5 days/week	NA*
8. Diesel Fuel Supply	Fuel Inventory	Weekly	10 days
9. Turbine Steam Stop, Control Valves	Closure	Monthly****	45 days****
10. Cable Tunnel Ventilation Fans	Functioning	Monthly	45 days
11. Control Room and Fuel Handling Building Filtration System	Charcoal Filter Pressure Drop Test < 5 inches of water visual inspection Freon - 112 (or equivalent) test \geq 99.5% at ambient conditions	Each refueling shutdown prior to refueling operation***	**

FREQUENCIES FOR EQUIPMENT TESTS

	<u>Check</u>	<u>Frequency</u>	<u>Maximum Time Between Tests</u>
12. Containment Air Filtration Systems	Visual Inspection	Each refueling***	**
	Pressure Drop Test < 5 inches of water	Each refueling***	**
	Charcoal coupons: iodine and ignition temperature 50% removal for methyl iodine, no ignition below 300° C.	Each refueling	**
	HEPA filters DOP ≥ 99% efficiency	Each refueling***	**

*NA - Not Applicable

**See Specification 1.9.

***Or at anytime work on the filters could alter their integrity

****This test may be waived during end-of-cycle operation when reactor coolant boron concentration is equal to or less than 150 ppm, due to operational limitations.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

Introduction

By letter dated June 4, 1979, Consolidated Edison Company of New York, Inc. (the licensee) requested a change to the Indian Point Unit 2 Technical Specifications. This change would extend the current monthly surveillance period for the turbine control and stop valves (T.S. Table 4.1-3 item 9) by up to two months at end of cycle by waiving the closure tests when boron concentration is less than 150 ppm.

Discussion

Operability of the turbine control and stop valves is checked monthly to assure the capability to control turbine speed in case of an unexpected loss of load. Failure of these valves could result in a failure of the turbine rotor and damage due to resultant missiles.

For end of cycle operations, performance of the closure tests leads to operational problems. Power must be reduced to less than 35%, which makes it difficult to maintain the axial flux difference within Technical Specification limits. The low boron concentration also requires processing of large amounts of reactor coolant to overcome xenon peaking and to return to full power. The licensee has requested a waiver of the test for end of cycle operations only.

Evaluation

Steam admission to the turbine is controlled by two valves in series; (1) the governing control valve and (2) the turbine stop valve. The stop valves are closed in response to a turbine overspeed trip signal, as well as other abnormal operating conditions. The control valves close on a rapid increase in turbine speed, in response to a load limit device, or when the stop valves are tripped. This electrical system is set to produce a trip on turbine overspeed at 106% of rated speed. Backup speed control is furnished by a mechanical

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overspeed mechanism set to trip at approximately 106% of rated speed. This redundancy in both valves and controls provides reasonable assurance that turbine speed control will be maintained.

The operating history of these valves includes more than 40 reactor and turbine trips, many trip signals during pre-operational and startup testing, and 23 other monthly tests since 1973. In all instances, the turbine stop and control valves have operated properly. This history of trouble-free performance provides added assurance of the dependability of these valves and the control systems.

The proposed technical specification change will allow the closure tests to be waived only at end-of-cycle when boron concentration is less than 150 ppm. Based on expected boron letdown rates of at least 80 ppm/month, this allows no more than a two month extension.

Based on the above, we consider the waiver of the turbine control and stop valve closure tests at end-of-cycle when the boron concentration is less than 150 ppm to be acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: June 7, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-247CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 54 to Facility Operating License No. DPR-26 issued to Consolidated Edison Company of New York, Inc. (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 2 (the facility) located in Buchanan, Westchester County, New York. The amendment is effective as of the date of issuance.

The amendment revises the Technical Specification to permit waiving the turbine stop and control valve testing at end of cycle.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated June 4, 1979, (2) Amendment No. 54 to License No. DPR-26, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 7th day of June.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors