

January 27, 1986

Docket No. 50-247

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M. Slosson

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PAD-3 Rdg

H. Thompson

C. Vogan

OELD

E. Jordan

J. Partlow

W. Jones

ACRS 10

LFMB

Mr. John D. O'Toole
Vice President
Nuclear Engineering and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. 108 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated June 7, 1985

The amendment revises the Technical Specifications to change the limiting conditions for operation (LCO's) for containment cooling and Iodine Removal Systems and associated containment isolation provisions. The amendment also contains editorial changes for consistency with the language used in other Indian Point 2 Technical Specifications.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/MSlosson

Marylee M. Slosson, Project Manager
PWR Project Directorate #3
Division of PWR Licensing-A

Enclosures:

- 1. Amendment No.108 to DPR-26
- 2. Safety Evaluation

cc: w/enclosures
See next page

*SEE PREVIOUS WHITE FOR CONCURRENCE

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CVogan MSlosson
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QEDD D/PAD-3
M. Karman *Syarga*
01/18/86 01/21/86
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Docket No. 50-247

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Mr. John D. O'Toole
Vice President
Nuclear Engineering and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

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OPA LFMB

Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated June 7, 1985

The amendment revises the Technical Specifications to change the limiting conditions for operation (LCO's) for containment cooling and Iodine Removal Systems and associated containment isolation provisions. The amendment also contains editorial changes for consistency with the language used in other of the Indian Point 2 Technical Specifications. In addition, the change eliminates the requirement to test redundant components daily when a component is inoperable.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager
PWR Project Directorate #3
Division of PWR Licensing-A

Enclosures:

- 1. Amendment No. to DPR-26
- 2. Safety Evaluation

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Mr. John D. O'Toole
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

cc:

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, New York 10511

Michael Blatt
Director Regulatory Affairs
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenues
Buchanan, New York, 10511

Robert L. Spring
Nuclear Licensing Engineer
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 38
Buchanan, New York 10511

Brent L. Brandenburg
Assistant General Counsel
Consolidated Edison Company
of New York, Inc.
4 Irving Place - 1822
New York, New York 10003

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Carl R. D'Alvia, Esquire
Attorney for the Village of
Buchanan, New York
395 South Riverside Avenue
Croton-on-Hudson, New York 10520

Mr. Jay Dunkleberger
Office of Policy Analysis
and Planning
York State Energy Office
Building 2, Empire State Plaza
Albany, New York 12223

Director, Technical Development
Programs
State of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Mr. Peter Kokolakis, Director
Nuclear Licensing
New York Power Authority
123 Main Street
White Plains, New York 10601

Mr. Murray Selman
Vice President, Nuclear Power
Consolidated Edison Company of
New York, Inc.
Broadway and Bleakley Avenues
Buchanan, New York 10511

Mr. Frank Matra
Resident Construction Manager
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenues
Buchanan, New York 10511

Ezra I. Bialik
Assistant Attorney General
Environmental Protection Bureau
New York State Department of Law
2 World Trade Center
New York, New York 10047



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.108
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated June 7, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

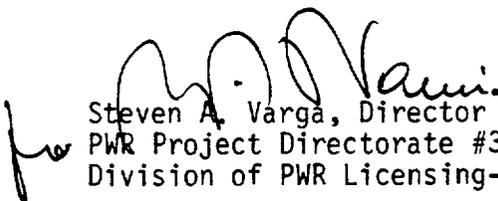
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.108 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Director
PWR Project Directorate #3
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 27, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 108 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

3.3-3

Table 3.6-1

Table 4.4-1 (pages 7-10, 14)

Insert Pages

3.3-3

Table 3.6-1

Table 4.4-1 (pages 7-10, 14)

- e. One channel of heat tracing may be out of service for 48 hours.
 - f. One refueling water storage tank low level alarm may be inoperable for up to 7 days provided the other low level alarm is operable.
3. When RCS temperature is less than or equal to 295°F, the requirements of Table 3.1.A-2 regarding the number of safety injection (SI) pumps allowed to be energized shall be adhered to.

B. Containment Cooling and Iodine Removal Systems

- 1. The reactor shall not be made critical unless the following conditions are met:
 - a. The spray additive tank contains not less than 4000 gallons of solution with a sodium hydroxide concentration of not less than 30% by weight.
 - b. The five fan cooler-charcoal filter units and the two spray pumps, with their associated valves and piping, are operable.
- 2. During power operation, the requirements of 3.3.B.1 may be modified to allow any one of the following components to be inoperable. If the system is not restored to meet the requirements of 3.3.B.1 within the time period specified, the reactor shall be placed in the hot shutdown condition utilizing normal operating procedures. If the requirements of 3.3.B.1 are not satisfied within an additional 48 hours, the reactor shall be placed in the cold shutdown condition utilizing normal operating procedures.
 - a. One fan cooler unit may be inoperable during normal reactor operation for a period not to exceed 7 days provided both containment spray pumps are operable.
 - b. One containment spray pump may be inoperable during normal reactor operation, for a period not to exceed 24 hours, provided the five fan cooler units and the remaining containment spray pump are operable.
 - c. Any valve required for the functioning of the system during and following accident conditions may be inoperable provided it is restored to operable status within 7 days or 24 hours for the fan cooler or containment spray systems respectively, and all valves in the system that provide the duplicate function are operable.

TABLE 3.6-1

NON-AUTOMATIC CONTAINMENT ISOLATION VALVES OPEN CONTINUOUSLY
OR INTERMITTENTLY FOR PLANT OPERATION

3418	851A	SWN-44-5-A or B(1)	1814B
3419	850A	SWN-51-5(1)	1814C
		SWN-44-1-A or B(1)	
4136	851B	SWN-51-1(1)	5018
			5019
744	850B	SWN-44-2-A or B(1)	5020
		SWN-51-2(1)	
888A	859A	SWN-44-3-A or B(1)	5021
888B	859C	SWN-51-3(1)	5022
958	863		5023
959	3416	SWN-44-4-A or B(1)	5024
990D	3417	SWN-51-4(1)	5025
1870	5459	SWN-71-5-A or B(1)	E-2
743	753H	SWN-71-1-A or B(1)	E-1
732	753G	SWN-71-2-A or B(1)	E-3
885A	SWN-41-5-A or B(1)	SWN-71-3-A or B(1)	E-5
885B	SWN-42-5	SWN-71-4-A or B(1)	MW-17
			MW-17-1
205	SWN-43-5	SA-24	85C
226	SWN-41-1-A or B(1)	SA-24-1	85D
227	SWN-42-1	PCV-1111-1	95C
250A	SWN-43-1	PCV-1111-2	95D
4925	SWN-41-2-A or B(1)	580A	
250B	SWN-42-2	580B	
4926	SWN-43-2	UH-43	
250C	SWN-41-3-A or B(1)	UH-44	
4927	SWN-42-3	990A	
250D	SWN-43-3	990B	
4928	SWN-41-4-A or B(1)	1814A	
869A	SWN-42-4		
878A	SWN-43-4		
869B			

(1) Either A or B valve(s) may serve as the required containment isolation valve(s) for the SWN-41, SWN-44 and SWN-71 series. Designation of the B valve(s) in the SWN-44 series requires the codesignation of the SWN-51 valve(s) associated with the penetration(s) as an additional required containment isolation valve(s).

TABLE 4.4-1 (Page 7 of 14)
CONTAINMENT ISOLATION VALVES

Valve No.	System ⁽¹⁾	Test Fluid ⁽²⁾	Minimum Test Pressure (PSIG)
SWN-41-5-A	Cont. Fan Cooler-Ser. Wtr.	Water ⁽⁶⁾	52
SWN-41-5-B	" " " " "	Water ⁽⁶⁾	52
SWN-43-5	" " " " "	Water ⁽⁶⁾	52
SWN-42-5	" " " " "	Water ⁽⁶⁾	52
SWN-41-1-A	" " " " "	Water ⁽⁶⁾	52
SWN-41-1-B	" " " " "	Water ⁽⁶⁾	52
SWN-43-1	" " " " "	Water ⁽⁶⁾	52
SWN-42-1	" " " " "	Water ⁽⁶⁾	52
SWN-41-2-A	" " " " "	Water ⁽⁶⁾	52
SWN-41-2-B	" " " " "	Water ⁽⁶⁾	52

Amendment No. 108

TABLE 4.4-1 (Page 8 of 14)
CONTAINMENT ISOLATION VALVES

Valve No.	System ⁽¹⁾	Test Fluid ⁽²⁾	Minimum Test Pressure (PSIG)
SWN-43-2	Cont. Fan Cooler-Ser. Wtr.	Water ⁽⁶⁾	52
SWN-42-2	" " " " "	Water ⁽⁶⁾	52
SWN-41-3-A	" " " " "	Water ⁽⁶⁾	52
SWN-41-3-B	" " " " "	Water ⁽⁶⁾	52
SWN-43-3	" " " " "	Water ⁽⁶⁾	52
SWN-42-3	" " " " "	Water ⁽⁶⁾	52
SWN-41-4-A	" " " " "	Water ⁽⁶⁾	52
SWN-41-4-B	" " " " "	Water ⁽⁶⁾	52
SWN-43-4	" " " " "	Water ⁽⁶⁾	52
SWN-42-4	" " " " "	Water ⁽⁶⁾	52
SWN-44-5-A	" " " " "	Water ⁽⁶⁾	52
SWN-44-5-B	" " " " "	Water ⁽⁶⁾	52
SWN-51-5	" " " " "	Water ⁽⁶⁾	52
SWN-44-1-A	" " " " "	Water ⁽⁶⁾	52
SWN-44-1-B	" " " " "	Water ⁽⁶⁾	52
SWN-51-1	" " " " "	Water ⁽⁶⁾	52

TABLE 4.4-1 (Page 9 of 14)
CONTAINMENT ISOLATION VALVES

Valve No.	System ⁽¹⁾	Test Fluid ⁽²⁾	Minimum Test Pressure (PSIG)
SWN-44-2-A	Cont. Fan Cooler-Ser. Wtr.	Water ⁽⁶⁾	52
SWN-44-2-B	" " " " "	Water ⁽⁶⁾	52
SWN-51-2	" " " " "	Water ⁽⁶⁾	52
SWN-44-3-A	" " " " "	Water ⁽⁶⁾	52
SWN-44-3-B	" " " " "	Water ⁽⁶⁾	52
SWN-51-3	" " " " "	Water ⁽⁶⁾	52
SWN-44-4-A	" " " " "	Water ⁽⁶⁾	52
SWN-44-4-B	" " " " "	Water ⁽⁶⁾	52
SWN-51-4	" " " " "	Water ⁽⁶⁾	52
SWN-71-5-A	" " " " "	Water ⁽⁶⁾	52
SWN-71-5-B	" " " " "	Water ⁽⁶⁾	52
SWN-71-1-A	" " " " "	Water ⁽⁶⁾	52
SWN-71-1-B	" " " " "	Water ⁽⁶⁾	52
SWN-71-2-A	" " " " "	Water ⁽⁶⁾	52
SWN-71-2-B	" " " " "	Water ⁽⁶⁾	52
SWN-71-3-A	" " " " "	Water ⁽⁶⁾	52
SWN-71-3-B	" " " " "	Water ⁽⁶⁾	52

TABLE 4.4-1 (Page 10 of 14)
CONTAINMENT ISOLATION VALVES

Valve No.	System ⁽¹⁾	Test Fluid ⁽²⁾	Minimum Test Pressure (PSIG)
SWN-71-4-A	Cont. Fan Cooler-Ser. Wtr.	Water ⁽⁶⁾	52
SNW-71-4-B	" " " " "	Water ⁽⁶⁾	52
SA-24	Service Air to Cont.	Water ⁽⁴⁾	52
SA-24-1	" " " " "	Water ⁽⁴⁾	52
580A	Dead Weight Tester	Gas	47
580B	" " " " "	Gas	47
UH-43	Auxiliary Steam System	Water ⁽⁴⁾	52
UH-44	" " " " "	Water ⁽⁴⁾	52
MW-17	City Wtr. to Cont.	Water ⁽⁴⁾	52
MW-17-1	" " " " "	Water ⁽⁴⁾	52
1170	Cont. Purge System	Gas ⁽⁷⁾	47
1171	" " " " "	Gas ⁽⁷⁾	47
1172	" " " " "	Gas ⁽⁷⁾	47
1173	" " " " "	Gas ⁽⁷⁾	47

TABLE 4.4-1 (Page 14 of 14)
CONTAINMENT ISOLATION VALVES

Valve No.	System ⁽¹⁾	Test Fluid ⁽²⁾	Minimum Test Pressure (PSIG)
95A	Equipment Airlock	Gas	47
95B	" "	Gas	47
95C	" "	Gas ⁽⁷⁾	47
95D	" "	Gas ⁽⁷⁾	47
4399	Sample Return to Cont. Sump.	Water ⁽⁴⁾	52
5132	" "	Water ⁽⁴⁾	52

Notes:

1. System description in which valve is located.
2. Gas Test Fluid indicates either nitrogen or air as test medium.
3. Testable only when at cold shutdown.
4. Isolation Valve Seal Water System.
5. Sealed by Residual Heat Removal System fluid.
6. Sealed by Service Water System. Either A or B valve(s) may serve as the required containment isolation valve(s) for the SWN-41, SWN-44 and SWN-71 series. Designation of the B valve(s) in the SWN-44 series requires the codesignation of the SWN-51 valve(s) associated with the penetration(s) as an additional required containment isolation valve(s).
7. Sealed by Weld Channel and Penetration Pressurization System.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 108 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

Introduction

By letter dated June 7, 1985, Consolidated Edison Company (the licensee) submitted a proposed amendment to the Technical Specification (Appendix A of Operating License No. DPR-26) for Indian Point Unit 2, to 1. Lengthen the allowed out-of-service time for containment fan cooling units; and 2. To permit the use of existing backup valving as alternate containment isolation valves.

1. Proposed Change to Allow Outage Time for Containment Fan Cooling Units 23, 24 and 25.

Evaluation and Discussion

Currently, Technical Specification 3.3.B.2.a limits continued power operation to either 24 hours, if fan cooler 23, 24 or 25 is out of service, or to seven days if fan cooler 21 or 22 is out of service, provided the containment spray system is operable. The differing out-of-service times are based upon the functional arrangement of the five fan cooler units and two containment spray pumps electrical configuration with the three diesel generator buses. The minimum set of safeguards equipment necessary to meet design basis containment heat removal objectives consists of three fan cooler units and one containment spray pump. The licensee contends, however, that the present 24-hour allowed out-of-service time for fan cooler units 23, 24 and 25 results in corrective actions which, while fully acceptable from a safety standpoint, are less than optimum from a practical operational standpoint; i.e., the 24-hour allowed outage time allows only isolation of tube banks rather than identification and isolation of the faulty tube. With the present 24-hour allowed outage time, maintenance designed to identify and isolate a faulty tube would require a plant cooldown and subsequent heatup, an unnecessary challenge to plant safety systems.

We have reviewed this proposed extension of the allowed outage times for fan cooler units 23, 24 and 25 and conclude that, although the safety margin for the design base accident will be decreased, the remaining equipment will be sufficient to keep containment peak pressure less than the design pressure of containment. Adoption of this amendment will not result in an increase in the probability of the Design Basis Accident nor will it create the possibility of a new or different kind of accident.

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2. Proposed Change to Containment Isolation Valve Provisions

The proposed changes to Table 3.6-1 and 4.4-1 of the Technical Specifications would permit the use of existing installed backup valving in the service water supply and return lines associated with containment fan cooling units as alternate containment isolation valves when so designated by plant management. By design, double boundary containment isolation requirements for these lines are satisfied by the closed system boundary inside containment and a containment isolation valve outside containment. In the event that a containment isolation valve becomes inoperable, this proposed change would permit redesignating the alternate containment isolation valve as the "required" containment isolation valve in the line, provided the surveillance requirement for the alternate valve has been satisfied.

We have reviewed this amendment and agree with the licensee that containment isolation requirements continue to be satisfied under this proposed change. We have also concluded that the proposed change will not involve a significant increase in the probability of a new or different kind of accident from any previously evaluated, nor involve a significant reduction in a margin of safety.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 27, 1986

Principal Contributor:

B. Hillman