

Docket No. 50-247

February 29, 1984
Mr. John D. O'Toole
Vice President
Nuclear Engineering and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

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Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No.87 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated February 14, 1983.

The amendment modifies the Technical Specifications to require a minimum of 23 feet of water above the reactor vessel flange during fuel handling operations.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY

Roger L. Pedersen, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

- 1. Amendment No.87 to DPR-26
- 2. Safety Evaluation

cc: w/enclosures
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script, appearing to read "Roger L. Pedersen".

Roger L. Pedersen, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No.87 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 87
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated February 14, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

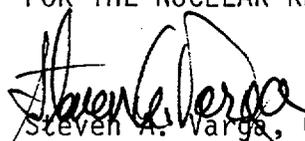
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 87, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 29, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 87 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

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3.8-3

Insert Page
3.8-3

fuel (72 assemblies or less) is to be discharged from the reactor, those assemblies in excess of one region shall not be discharged before a continuous interval of 400 hours has elapsed after shutdown.

10. The minimum water level above the top of reactor pressure vessel flange shall be at least 23 feet (El. 92'0") whenever movement of spent fuel is being made.
11. A dead-load test shall be successfully performed on the fuel storage refueling building crane before fuel movement begins. The load assumed by the refueling crane for this test must be equal to or greater than the maximum load to be assumed by the refueling crane during the refueling operation. A through visual inspection of the refueling crane shall be made after the dead load test and prior to fuel handling.
12. The fuel-handling building charcoal filtration system must be operating whenever spent fuel movement is being made unless the spent fuel has had a continuous 35-day decay period.
13. A licensed senior reactor operator shall be at the site and designated in charge of the operation whenever changes in core geometry are taking place.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 87 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

Introduction

By letter dated August 15, 1980 from T. M. Novak to all Westinghouse PWR Licensees, Consolidated Edison (Con. Ed.) was requested to revise the Indian Point Unit 2 (IP2) Technical Specifications (TS) in consonance with the recent revisions to the Westinghouse Standard Technical Specifications (NUREG-0452). These revisions required at least 23 feet of water over the top of the reactor pressure vessel flange during movement of fuel assemblies or control rods.

In response to the August 1980 letter, Con. Ed. requested, by letter dated February 14, 1983, changes to the IP2 Technical Specifications requiring 23 feet of water above the reactor flange during refueling operations.

The February submittal from Con. Ed. contained requests for TS changes dealing with several other issues. This SER addresses only the water above the reactor issue. The other TS change requests will be the subject of separate licensing actions.

Evaluation

The restriction of 23 feet minimum of water above the core assembly in the current TS ensures that sufficient water depth is available to remove 99% of the assumed 10% iodine gas activity released from the rupture of an irradiated fuel assembly. During defueling operations however, a fuel assembly may need to be lifted in excess of 23 feet to clear the vessel flange. Consequently, part of the fuel assembly could be exposed if the depth of water over the core assembly did not exceed 23 feet. The proposed change to require 23 feet of water above the vessel flange provides sufficient depth of water to prevent inadvertent exposure of a fuel assembly.

The licensee's proposal to limit the water level to at least 23 feet above the reactor vessel flange during refueling operations meets the intent of the August 15, 1980 Novak letter and is in consonance with the Standard Technical Specifications. Therefore the proposed change is acceptable to the staff.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made

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this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 29, 1984

Principal Contributors:

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