UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 16, 1993

Docket No. 50-247

Mr. Stephen B. Bram
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

Dear Mr. Bram:

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING UNIT NO. 2 (TAC NO. M85141)

The Commission has issued the enclosed Amendment No. 166 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated November 25, 1992, as supplemented by letter dated February 5, 1993.

The amendment revises surveillance intervals for Process Radiation Monitors, Area Radiation Monitors, the Main Steam Line Radiation Monitors, the Auxiliary Feedwater System Initiating Logic, the Main Steam Safety Valves Setpoints, and the Toxic Gas Detection System Monitors to accommodate a 24-month refueling cycle. These revisions are being made in accordance with the guidance provided by Generic Letter 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle."

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Francis J. Williams, Jr., Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 166 to DPR-26
2. Safety Evaluation

cc w/enclosures:
See next page

DF01

cc:

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Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

DATED: December 16, 1993

AMENDMENT NO. 166 TO FACILITY OPERATING LICENSE NO. DPR-26-INDIAN POINT UNIT 2

Docket File
NRC & Local PDRs
PDI-1 Reading
S. Varga, 14/E/4
J. Calvo, 14/A/4
R. Capra
C. Vogan
F. Williams
OGC
D. Hagan, 3302 MNBB
G. Hill (2), P1-22
C. Grimes, 11/F/23
ACRS (10)
OPA
OC/LFDCB
PD plant-specific file
C. Cowgill, Region I
L. Cunningham, 10/D/4

cc: Plant Service list

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 166
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Consolidated Edison Company of of New York, Inc. (the licensee) dated November 25, 1992, as supplemented by letter dated February 5, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 166, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 16, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 166

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages	Insert Pages
Table 4.1-1 (page 2 of 7)	Table 4.1-1 (page 2 of 7)
Table 4.1-1 (page 6 of 7)	Table 4.1-1 (page 6 of 7)
Table 4.1-3 (page 1 of 1)	Table 4.1-3 (page 1 of 1)
4.5-7	4.5-7
4.8-1	4.8-1
Table 4.10-2 (page 1 of 2)	Table 4.10-2 (page 1 of 2)
Table 4.10-4 (page 1 of 2)	Table 4.10-4 (page 1 of 2)
Table 4.10-4 (page 2 of 2)	Table 4.10-4 (page 2 of 2)

Table 4.1-1

Minimum Frequencies for Checks, Calibrations and Tests of Instrument Channels

	Channel Description	Check	Calibrate	Test	Remarks
10.	Rod Position Bank Counters	S	N.A.	N.A.	With analog rod position
11.	Steam Generator Level	S	R#	Q	
12.	Charging Flow	N.A.	R#	N.A.	
13.	Residual Heat Removal Pump Flow	N.A.	R#	N.A.	
14.	Boric Acid Tank Level	W	R	N.A.	Bubbler tube rodded during calibration
15.	Refueling Water Storage Tank Level	W	R	N.A.	
16.	DELETED				
17.	Volume Control Tank Level	N.A.	R	N.A.	
18a.	Containment Pressure	D	R	Q	Wide Range
18b.	Containment Pressure	S	R	Q	Narrow Range
18c.	Containment Pressure (PT-3300,PT-3301)	M	R	N.A.	High Range
19.	Process Radiation Monitoring System	D	R#	M	
19a.	Area Radiation Monitoring System	D	R#	M	
19b.	Area Radiation Monitoring System (VC)	D	R#	M	

Table 4.1-1

Minimum Frequencies for Checks, Calibrations and Tests of Instrument Channels

	Channel Description	Check	Calibrate	Test	Remarks
38.	Wide Range Plant Vent Noble Gas Effluent Monitor (R-27)	S	R	N.A.	
39.	Main Steam Line Radiation Monitor (R-28, R-29, R-30, R-31)	S	R#	N.A.	
40.	High Range Containment Radiation Monitor (R-25, R-26)	S	R*	N.A.	
41.	Containment Hydrogen Monitor	Q	Q**	N.A.	

* Acceptable criteria for calibration are provided in Table II.F-13 of NUREG-0737.

** Calibration will be performed using calibration span gas.

Table 4.1-3

Frequencies for Equipment Tests

		Check	Frequency	Maximum Time Between Tests
1.	Control Rods	Rod drop times of all control rods	Refueling # Interval	*
2.	Control Rods	Movement of at least 10 steps in any one direction of all control rods	Every 31 days during reactor critical operations	*
3.	Pressurizer Safety Valves	Setpoint	Refueling # Interval	*
4.	Main Steam Safety Valves	Setpoint	Refueling # Interval	*
5.	Containment Isolation System	Automatic Actuation	Refueling Interval	*
6.	Refueling System Interlocks	Functioning	Each refueling shutdown prior to refueling operation	Not Applicable
7.	Diesel Fuel Supply	Fuel Inventory	Weekly	10 days
8.	Turbine Steam Stop Control Valves	Closure	Monthly**	45 days**
9.	Cable Tunnel Ventilation Fans	Functioning	Monthly	45 days

*, See Specification 1.9.

** This test may be waived during end-of-cycle operation when reactor coolant boron concentration is equal to or less than 150 ppm, due to operational limitations.

6. After each complete or partial replacement of a charcoal adsorber bank, by verifying that the charcoal adsorbers remove greater than or equal to 99.95% of a halogenated hydrocarbon refrigerant test gas when they are tested in-place in accordance with ANSI N510-1975 while operating the system at ambient conditions and at a flow rate of 1840 cfm ±10%.

7. Each toxic gas detection system shall be demonstrated operable by performance of a channel check at least once per day, a channel test at least once per 31 days and a channel calibration at least once each Refueling Interval (#).

F. FUEL STORAGE BUILDING AIR FILTRATION SYSTEM

The fuel storage building air filtration system specified in Specification 3.8 shall be demonstrated operable:

1. At least once per 31 days by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the system operates for at least 15 minutes.

2. At each refueling, prior to refueling operations, or (1) after any structural maintainance on the HEPA filter or charcoal adsorber housings, or (2) at any time painting, fire or chemical releases could alter filter integrity by:

   a. verifying a system flow rate at ambient conditions of 20,000 cfm ±10% during system operation when tested in accordance with ANSI N510-1975.

   b. verifying that the system satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a, C.5.c and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978, at ambient conditions and at a flow rate of 20,000 cfm ±10%.

4.8 AUXILIARY FEEDWATER SYSTEM

Applicability

Applies to periodic testing requirements of the Auxiliary Feedwater System.

Objective

To verify the operability of the Auxiliary Feedwater System and its ability to respond properly when required.

Specifications

A. The following surveillance tests shall be performed at least once every Refueling Interval:

1. Verification of proper operation of auxiliary feedwater system components and initiating logic upon receipt of test signals for each mode of automatic initiation[1].

2. Verification of the capability of each auxiliary feedwater pump to deliver full flow to the steam generators[1].

B. The above tests shall be considered satisfactory if control board indication and subsequent visual observation of the equipment demonstrate that all components have operated properly.

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1. In this instance Refueling Interval is defined as R#.

Table 4.10-2

Radioactive Liquid Effluent Monitoring Instrumentation Surveillance Requirements

Instrument	Channel Check	Source Check	Channel Calibration	Channel Functional Test
1. GROSS RADIOACTIVITY MONITORS PROVIDING ALARM AND AUTOMATIC TERMINATION OF RELEASE				
a. Liquid Radwaste Effluent Line	D*	P	R(3)#	Q(1)
b. Steam Generator Blowdown Effluent Line	D*	M	R(3)#	Q(1)
2. GROSS BETA OR GAMMA RADIOACTIVITY MONITORS PROVIDING ALARM BUT NOT PROVIDING AUTOMATIC TERMINATION OF RELEASE				
a. Service Water System Effluent Line	D*	M	R(3)#	Q(2)
b. Unit 1 Secondary Boiler Blowdown Effluent Line	D*	M	R(3)#	Q(2)
3. FLOW RATE MEASUREMENTS DEVICES				
a. Liquid Radwaste Effluent Line	D(4)	N.A.	R	Q
b. Steam Generator Blowdown Effluent Line	D(4)	N.A.	R	Q
4. TANK LEVEL INDICATING DEVICES***				
a. 13 Waste Distillate Storage Tank	D**	N.A.	R	Q
b. 14 Waste Distillate Storage Tank	D**	N.A.	R	Q
c. Primary Water Storage Tank	D**	N.A.	R	Q
d. Refueling Water Storage Tank	D**	N.A.	R	Q
e. 21 Monitor Tank	D**	N.A.	R	Q
f. 22 Monitor Tank	D**	N.A.	R	Q
g. 23 Monitor Tank	D**	N.A.	R	Q

* During releases via this pathway

** During liquid additions to the tank

*** Tanks included in this specification are those outdoor tanks that are not surrounded by liners, dikes, or walls capable of holding the tank contents and do not have tank overflow and surrounding area drains connected to the liquid radwaste treatment system.

Table 4.10-4

Radioactive Gaseous Effluent Monitoring Instrumentation Surveillance Requirements

Instrument	Channel Check	Source Check	Channel Calibration	Channel Functional Test	Modes In Which Surveillance Required
1. WASTE GAS HOLDUP SYSTEM					
a. Noble Gas Activity Providing Alarm	D	M	R(3)#	Q(2)	*
2. WASTE GAS HOLDUP SYSTEM EXPLOSIVE GAS MONITORING SYSTEM					
a. Hydrogen Monitor	D	N.A.	Q(4)	M	**
b. Hydrogen or Oxygen Monitor	D	N.A.	Q(5)	M	**
3. CONDENSER EVACUATION SYSTEM					
a. Noble Gas Activity	D	M	R(3)	Q(2)	*
4. PLANT VENT					
a. Noble Gas Activity Monitor	D	M	R(3)	Q(2)	*
b. Iodine Sampler	W	N.A.	N.A.	N.A.	*
c. Particulate Sampler	W	N.A.	N.A.	N.A.	*
d. Flow Rate Monitor	D	N.A.	R	N.A.	*
e. Sampler Flow Rate Monitor	D	N.A.	R	N.A.	*
5. STACK VENT					
a. Noble Gas Activity Monitor	D	P	R(3)	Q(1)	*
b. Iodine Sampler	W	N.A.	N.A.	N.A.	*
c. Particulate Sampler	W	N.A.	N.A.	N.A.	*
d. Flow Rate Monitor	D	N.A.	R	N.A.	*
e. Sampler Flow Rate Monitor	D	N.A.	R	N.A.	*

* Surveillance is required at all times except when monitor has been removed from service in accordance with Table 3.9-2.

** During waste gas holdup system operation (treatment for primary system off-gasses).

Table 4.10-4

Radioactive Gaseous Effluent Monitoring Instrumentation Surveillance Requirements

Table Notation

(1) The channel functional test shall also demonstrate that automatic isolation of this pathway and control room alarm annunciation occurs if the following conditions exist:

1. Instrument indicates measured levels above the alarm/trip setpoint

(2) The channel functional test shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:

1. Instrument indicates measured levels above the alarm setpoint.

2. Instrument controls not set in operate mode.

(3) Radioactive Calibration Standards used for channel calibrations shall be traceable to the National Bureau of Standards or an aliquot of calibration gas shall be analyzed with instrumentation which is calibrated with NBS traceable standards (standards from suppliers who participate in measurement assurance activities with NBS are acceptable).

(4) The channel calibration shall include the use of standard gas samples containing:

1. less than or equal to two volume percent hydrogen, and

2. greater than or equal to four volume percent hydrogen.

(5) The channel calibration shall include the use of standard gas samples containing:

1. less than or equal to two volume percent oxygen, and

2. greater than or equal to two volume percent oxygen.

UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 166 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

1.0 INTRODUCTION

By letter dated November 25, 1992, as supplemented by letter dated February 5, 1993, the Consolidated Edison Company of New York (the licensee) submitted requests for changes to the Indian Point Nuclear Generating Unit No. 2 Technical Specifications (TS). The requested changes are a follow-up to License Amendment No. 159, issued on December 10, 1992, which changed the TS Section 1.0, Definitions, to accommodate a 24-month fuel cycle and which extended test intervals for specific surveillance tests. The requested changes in this proposal would extend the surveillance intervals to 24 months for Process Radiation Monitors, Area Radiation Monitors, the Main Steam Line Radiation Monitors, the Auxiliary Feedwater System Initiating Logic, the Main Steam Safety Valves Setpoints, and the Toxic Gas Detection System Monitors. The changes requested by the licensee are related to a 24-month fuel cycle and are in accordance with Generic Letter (GL) 91-04, "Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle."

2.0 EVALUATION

The licensee has proposed specific surveillance interval extensions to accommodate a 24-month fuel cycle. Process radiation monitors included in the request are listed as follows:

- Radiation Monitor R-47, Component Cooling Loop
- Radiation Monitors R-41 and R-42, Containment Atmosphere
- Radiation Monitors R-48 and R-54, Liquid Radwaste Effluent
- Radiation Monitor R-49, Steam Generator Blowdown
- Radiation Monitor R-52, Steam Generator Blowdown Purification Cooling Water

- Radiation Monitor R-51, Steam Generator Blowdown after Purification
- Radiation Monitor R-59, Condensate Return to House Boilers
- Radiation Monitors R-55A, B, C and D, Steam Generator Sampling
- Radiation Monitors R-46 and R-53, Service Water from Containment Fan Coolers
- Radiation Monitors R-39 and R-40, Service Water from Component Cooling Heat Exchangers
- Area Radiation Monitors
- Main Steam Line Radiation Monitors

The radiation monitors in the above list are not safety related and, therefore, do not have setpoints critical to plant safety, nor are their readings used in any calculations which require discrete accuracy. In all cases their function is to detect changes in levels of radiation at absolute values well below allowable limits so that instrument drift is relatively unimportant. Concentrations of specific radionuclides are determined by laboratory analysis of grab samples. There are no effects on plant operation, other than a requirement to take grab samples, caused by inoperability of these monitors. Most of the above monitors are relatively new instruments installed under the plant Radiation Monitoring Betterment Program and, therefore, there is limited historical data on surveillance performance. The licensee has reviewed the available data, however, and has concluded that there have been no discrepancies which would have rendered a monitor inoperable. In addition, all of the above monitors, with the exception of the Main Steam Line Radiation Monitors, are subjected to daily checks and monthly tests (in lieu of monthly tests, the Liquid Radwaste Monitors R-48 and R-54 are tested prior to each discharge). The Main Steam Line Radiation Monitors are checked each shift and each detector assembly includes a constant depleted uranium source giving a fixed readout. This feature takes the place of the usual electrically activated check source mechanism. Some monitors are also given quarterly functional tests. These checks and tests would detect gross abnormalities prior to the refueling calibrations. The licensee concludes that the increase of the surveillance interval from 18 to 24 months for these monitors would have no significant effect on safety.

The licensee has evaluated the effect of the increase in the surveillance intervals on safety and has concluded that the effect is small. The licensee has confirmed that historical plant maintenance and surveillance data do not invalidate this conclusion. The licensee also confirmed that the increase in surveillance intervals to accommodate a 24-month fuel cycle does not invalidate any assumption in the plant licensing basis. The staff reviewed

this information and agrees with the basis and conclusions contained therein. The staff finds that the proposed TS changes do not have a significant effect on safety and are, therefore, acceptable.

In addition to the radiation monitors, the licensee has proposed extending the interval for the following surveillances:

- Auxiliary Feedwater Initiating Logic
- Main Steam Safety Valves
- Toxic Gas Detection System

The Auxiliary Feedwater System supplies high-pressure feedwater to the steam generators to maintain a water inventory for decay heat removal. The system is used for normal startup and shutdown as well as for events leading to a loss of main feedwater. The refueling interval surveillance requires verification of proper operation of the initiating logic for the auxiliary feedwater system components upon receipt of test signals for each mode of operation. The licensee reviewed completed test procedures for the last four refuelings which covered a period in excess of 5 years. In one instance a test result did not meet the acceptance criteria. In 1986, a relay failed to pickup due to dirty contacts. This failure only affected the steam driven auxiliary feedwater pump and would not have prevented the auxiliary feedwater system from supplying more than the required amount of feedwater. The licensee concluded that no significant increase in probability or consequence of an accident would be incurred by extending the surveillance interval.

The Main Steam Safety Valves provide one means of discharging excess steam if the condenser heat sink is not available during a turbine trip. There are five code safety valves located on each of the four main steam lines outside the reactor containment upstream of the isolation and nonreturn valves. The total relieving capacity of the 20 main steam safety valves is 114 percent of the total secondary steam flow at 100 percent rated thermal power. Operability of the safety valves is verified each refueling shutdown. The licensee reviewed completed test procedures for the last four refuelings which covered a period in excess of 5 years. Five out of the 80 tests indicated "as-found" setpoints out of tolerance and not conservative. In two of these cases the deviations from the setpoints were only 1 psi and 4 psi and were not considered to be significant. In the remaining three cases the deviations were higher than the upper limits but in all cases were below the setpoints of the remaining two safety valves with higher setpoints. The licensee concluded that the historical data support the extension of the surveillance interval.

The Toxic Gas Detection system is designed to isolate the control room from outside air upon detection of toxic concentrations of the monitored gases in the control room ventilation system. The previous toxic gas monitors were replaced in May and August of 1992. The new monitoring system uses a unique

gas diffusion sensor which responds rapidly and is free of interferences from most common gases. The sensor does not lose sensitivity with prolonged absence of the toxic gas and is not poisoned by ambient gases or vapors. Since these are new monitors, there are no previous data to project drift characteristics. However, daily channel checks and monthly functional tests performed on these instruments would detect drift outside calibration limits permitting adjustment. The licensee concluded that the surveillance extension would have no effect on safety.

The licensee has evaluated the effect of the increase in the above surveillance intervals on safety and has concluded that the effect is small. The licensee has confirmed that historical plant maintenance and surveillance data do not invalidate this conclusion. The licensee also confirmed that the increase in surveillance intervals to accommodate a 24-month fuel cycle does not invalidate any assumption on the plant licensing basis. The staff reviewed this information and agrees with the conclusions contained therein. The staff finds that the proposed TS changes do not have a significant effect on safety and are, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 16219). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
F. Williams

Date: December 16, 1993

Docket No. 50-247

December 16, 1993

Mr. Stephen B. Bram
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

Dear Mr. Bram:

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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Francis J. Williams, Jr., Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 166 to DPR-26
2. Safety Evaluation

cc w/enclosures:
See next page

Distribution:
See attached sheet
*See previous concurrence

LA:PDI-1	PM:PDI-1	*PRPB	*OGC	PDI-1	
CVogan	FWilliams:smm	LCunningham		RACapra	
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