

November 4, 1986

Docket No. 50-247

Mr. Murray Selman
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
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Dear Mr. Selman:

SUBJECT: AMENDMENT NO. 115 TO THE INDIAN POINT NUCLEAR GENERATING
UNIT NO. 2 TECHNICAL SPECIFICATIONS

By letter dated August 7, 1986, Amendment 115 to Facility License DPR-26 for the Indian Point Nuclear Generating Unit No. 2 was issued. The amendment was issued with an administrative error. The amendment was issued with page 6-13 from your original submittal dated June 18, 1985. This page was revised, as requested by the NRC staff, by your April 18, 1986 letter. A copy of the corrected page is enclosed.

I apologize for any inconvenience this may have caused you.

Sincerely,

Marylee Slosson, Project Manager
Project Directorate #3
Division of PWR Licensing-A

Enclosure:
As stated

cc: See next page

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Mr. Murray Selman
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of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

cc:

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Ezra I. Bialik
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d. Quality Assurance Program for effluent and environmental monitoring using the guidance in Regulatory Guide 1.21, Revision 1, April 1974 and Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and any changes to them shall be reviewed and approved for implementation in accordance with a written administrative control procedure approved by the appropriate General Manager, with the concurrence of the Station Nuclear Safety Committee and the Nuclear Facilities Safety Committee. The administrative control procedure required by this specification shall, as a minimum, require that:

- a. Each proposed procedure/procedure change involving safety related components and/or operation of same receives a pre-implementation review by the SNSC except in case of an emergency.
- b. Each proposed procedure/procedure change which renders or may render the Final Safety Analysis Report or subsequent safety analysis reports inaccurate and those which involve or may involve potential unreviewed safety questions are approved by the SNSC prior to implementation.
- c. The approval of the Nuclear Facilities Safety Committee shall be sought if, following its review, the Station Nuclear Safety Committee, finds that the proposed procedure/procedure change either involves an unreviewed safety question or if it is in doubt as to whether or not an unreviewed safety question is involved.

6.8.3 A mechanism shall exist for making temporary changes and they shall only be made by approved management personnel in accordance with the requirements of ANSI 18.7-1972. The change shall be documented, and reviewed by the SNSC and approved by a General Manager within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

- a. A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:
 - (i) Training of personnel,
 - (ii) Procedures for sampling and analysis,
 - (iii) Provisions for maintenance of sampling and analysis equipment.

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