December 7, 2000 RC-00-0368



Document Control Desk U. S. Nuclear Regulatory Commission Washington, DC 20555

Gentlemen:

Subject:

Stephen A. Byrne Vice President Nuclear Operations 803.345.4622

South Carolina Electric & Gas Co. Virgil C. Summer Nuclear Station P. O. Box 88 Jenkinsville, South Carolina 29065

803.345.5209 803.635.1461 ot: VIRGIL C. SUMMER NUCLEAR STATION DOCKET NO. 50/395 OPERATING LICENSE NO. NPF-12 INTERIM REPORT PURSUANT TO 10 CFR PART 21 (SSH 2000-001)

South Carolina Electric & Gas (SCE&G) submits this letter in accordance with the requirements of 10 CFR 21.21(a)(2) as an interim report of an identified defect potentially associated with a substantial safety hazard.

On 10/12/00, at 0630 hours, visual inspection revealed small amounts of boron buildup on the weld between the vessel nozzle and the hot leg pipe. Within hours, the suspect area was cleaned and a dye penetrant (PT) examination of the pipe identified a 4 inch indication at the weld between the hot leg piping and the reactor vessel nozzle. The weld is located approximately 3 feet from the vessel in pipe near the nozzle. The indication was located about 17" from the top of the pipe. This pipe has a nominal inside diameter of 29 inches and is approximately 2.5 inches thick.

Subsequent ultrasonic examination from the inside diameter identified an axial flaw less than 3 inches long. The same examination determined that the original indication was not the source of the leak.

The issue resolution strategy consists of a root cause evaluation, repair of the weld, and restart justification. A group of industry experts in the areas of root cause, failure analysis, materials, welding, inspection, and fracture mechanics has been assembled to assist in the recovery effort.

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The section of piping containing the weld in question was shipped to Westinghouse for analysis at their Spartanburg, S. C. and Waltzmill, Pa. facilities as input to a root cause determination. The root cause evaluation will not be completed within the 60 day 10 CFR Part 21 evaluation period. This interim 10 CFR Part 21 report is being submitted to allow additional evaluation and cause determination by the Root Cause Team. The Root Cause Team anticipates that an initial root cause determination should be completed later this month. A final report on the 10 CFR Part 21 evaluation will be submitted within thirty days of the final Root Cause Analysis Report, expected in January 2001.

Should you have any questions, please call Mr. Jeffrey Pease at (803) 345-4124.

Very truly yours,

JWP/SAB/dr

C:

N. O. Lorick N. S. Carns T. G. Eppink R. J. White L. A. Reyes K. R. Cotton J. B. Knotts, Jr. NRC Resident Inspector G. G. Williams NSRC RTS (SSH 2000-001) File (818.18) DMS (RC-00-0368)