

June 17, 1987

Docket No. 50-247

Mr. Murray Selman
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
New York, New York 10511

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E. Butcher	ACRS (10)
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P. Swetland	

Dear Mr. Selman:

The Commission has issued the enclosed Amendment No. 120 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 11, 1987 (TAC 64961).

The amendment revises the Technical Specifications to incorporate station batteries nos. 23 and 24 under the same provisions as batteries nos. 21 and 22. The actual modification to Indian Point 2 to add the 2 batteries to the battery system was reviewed and accepted in NRC Safety Evaluation dated May 2, 1980.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager
Project Directorate I-1
Division of Reactor Projects 1/2

Enclosures:

1. Amendment No.120 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

* SEE PREVIOUS CONCURRENCE

PDI-1
CVogan*
5/4/87

PDI-1
MSlosson*
5/11/87

*MSlosson**
6-17-87

OGC-Beth
MKarman*
6/12/87

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6/17/87

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Region V (4)	

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PDI-1
CVogan*
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MSlosson*
5/10/87

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6/1/87~~

OGC Beth
M. Karman
6/12/87



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PDI-1
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GPA/PA	ARM/LFMB
Region V (4)	

Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. _____ to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 11, 1987.

The amendment revises the Technical Specifications to incorporate station batteries nos. 23 and 24 under the same provisions as batteries nos. 21 and 22. The actual modification to Indian Point 2 to add the 2 batteries to the battery system was reviewed and accepted in NRC Safety Evaluation dated May 2, 1980.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

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Marylee M. Slosson, Project Manager
Project Directorate I-1
Division of Reactor Projects 1/2

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cc: w/enclosures
See next page

PDI-1 *CV*
CVogan
5/4/87

PDI-1 *MS*
MSlosson
5/11/87

OGC-Beth
4/ /87

PDI-1
RCapra
4/ /87

ESB

5/1/87

Mr. Murray Selman
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 120
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated March 11, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 120, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects 1/2

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 17, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 120 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3.7-1	3.7-1
3.7-2	3.7-2
3.7-3	3.7-3
4.6-2	4.6-2

Applicability

Applies to the availability of electrical power for the operation of plant auxiliaries.

Objective

To define those conditions of electrical power availability necessary (1) to provide for safe reactor operation and (2) to provide for the continuing availability of engineered safety features.

Specification

- A. The reactor shall not be made critical without:
1. Two 138 kv lines to Buchanan fully operational.
 2. The 6.9 kv buses 5 and 6 energized from the 138 kv source.
 3. One 13.8 kv source fully operational and the 13.8/6.9 kv transformer available to supply 6.9 kv power.
 4. The four 480-volt buses 2A, 3A, 5A and 6A energized and the bus tie breakers between buses 5A and 2A and between buses 3A and 6A open.
 5. Three diesel generators operable with on-site supply of 19,000 gallons of fuel available in the individual storage tanks and 22,000 gallons of fuel available on-site other than the normal supply tanks.
 6. Station Batteries Nos. 21, 22, 23, & 24 and their associated battery chargers and d.c. distribution systems operable.

B. During power operation, the following components may be inoperable:

1. Power operation may continue for seven days if one diesel is inoperable provided the 138kv and the 13.8kv sources of off-site power are available and the remaining diesel generators are tested daily to ensure operability and the engineered safety features associated with these diesel generator buses are operable.
2. Power operation may continue for 24 hours, if the 138kv or the 13.8kv source of power is lost, provided the three diesel generators are operable. This operation may be extended beyond 24 hours provided the failure is reported to the NRC within the subsequent 24-hour period with an outline of the plans for restoration of off-site power.
3. If the 138kv power source is lost, in addition to satisfying the requirements of specification 3.7.B.2 above, the 6.9kv bus tie breaker control switches 1-5, 2-5, 3-6, and 4-6 in the CCR shall be placed in the "pull-out" position and tagged to prevent an automatic transfer of the 6.9KV buses 1,2,3 and 4.
4. One battery may be inoperable for 24 hours provided the other batteries and four battery chargers remain operable with one battery charger carrying the dc load of the failed battery's supply system.

C. Gas Turbine Generators:

1. At least one gas turbine generator (GT-1, GT-2, or GT-3) and associated switchgear and breakers shall be operable at all times.
2. A minimum of 54,200 gallons of fuel for the operable gas turbine generator shall be available at all times.
3. If the requirements of 3.7.C.1 or 3.7.C.2 cannot be met, then, within the next seven (7) days, either the inoperable condition shall be corrected or an alternate independent power system shall be established.
4. If the requirements of 3.7.C.3 cannot be satisfied, the reactor shall be placed in the hot shutdown condition utilizing normal operating procedures. If the requirements of 3.7.C.3 cannot be met within an additional 48 hours, the reactor shall be placed in the cold shutdown condition utilizing normal operating procedures.

The requirements of Specification 3.7.A may be modified for an emergency "Black Start" of the unit by using the requirements of either Specification 3.7.D.1 or 3.7.D.2 below:

1.
 - a. All 138 kv lines to Buchanan de-energized.
 - b. The 13.8 kv line de-energized.
 - c. The 6.9 kv buses 5 and 6 energized from the on-site gas turbine through the 13.8/6.9 kv transformer.
 - d. The four 480-volt buses 2A, 3A, 5A and 6A energized from the diesels and the tie breakers between buses 5A and 2A and between buses 3A and 6A open.
 - e. Three diesel generators operable with on-site supply of 19,000 gallons of fuel available in the individual storage tanks and 22,000 gallons of fuel available on-site other than the normal supply tanks and supplying 480-volt buses.
 - f. Station batteries Nos. 21, 22, 23, & 24 and their associated battery chargers and d.c. distribution systems operable.
 - g. The 480-volt tie breakers 52/2A, 52/3A, 52/5A, and 52/6A open.

2.
 - a. Establish 138 kv bus sections at Buchanan with at least 37 MW power (nameplate rating) from any combination of gas turbines at Buchanan and on-site.
 - b. Two 138 kv lines to Buchanan energized from the gas turbines with breakers to Millwood and Orange and Rockland open.
 - c. The 138 kv line to Buchanan operable and the 13.8/6.9 kv transformer available to supply 6.9 kv power.
 - d. The 6.9 kv buses energized from the 138 kv source.
 - e. The four 480-volt buses 2A, 3A, 5A and 6A energized and the bus tie breakers between buses 5A and 2A and between buses 3A and 6A open.
 - f. Three diesel generators operable with on-site supply of 19,000 gallons of fuel available in the individual storage tanks and 22,000 gallons of fuel available on-site other than the normal supply tanks.
 - g. Station batteries Nos. 21, 22, 23, & 24 and their associated battery chargers and d.c. distribution systems operable.

4. Each diesel generator shall be given a thorough inspection at least annually following the manufacturer's recommendations for this class of stand-by service.

The above tests will be considered satisfactory if the required minimum safeguards equipment operated as designed.

3. Diesel Fuel Tanks

A minimum oil storage of 41,000 gallons will be maintained at the station at all times.

C. Station Batteries (Nos. 21, 22, 23 & 24)

1. Every month the voltage of each cell, the specific gravity and temperature of a pilot cell in each battery and each battery voltage shall be measured and recorded.
2. Every 3 months each battery shall be subjected to a 24 hour equalizing charge, and the specific gravity of each cell, the temperature reading of every fifth cell, the height of electrolyte, and the amount of water added shall be measured and recorded.
3. At each time data is recorded, new data shall be compared with old to detect signs of abuse or deterioration.
4. At each refueling interval, each battery shall be subjected to a load test and a visual inspection of the plates.

D. Gas Turbine Generators

1. At monthly intervals, at least one gas turbine generator shall be started and synchronized to the power distribution system for a minimum of thirty (30) minutes with a minimum electrical output of 750 KW.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 120 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

INTRODUCTION

By letter dated March 11, 1987, the Consolidated Edison Company of New York submitted proposed changes to Technical Specifications, Sections 3.7 and 4.6 relating to batteries nos. 23 and 24 at the Indian Point Station, Unit 2. The proposed changes would include station batteries nos. 23 and 24, and their associated battery chargers and d.c. distribution systems, under the existing provisions for station batteries nos. 21 and 22. For example, one of the present Technical Specifications states:

The reactor shall not be made critical without Station Batteries Nos. 21 & 22 and their associated battery chargers and d.c. distribution systems operable.

The above would be changed to:

The reactor shall not be made critical without Station Batteries Nos. 21, 22, 23 & 24 and their associated battery chargers and d.c. distribution systems operable.

In effect, the present Technical Specification would be extended to include batteries nos. 23 and 24 and their associated battery chargers and d.c. distribution systems.

DISCUSSION AND EVALUATION

The licensee states that batteries nos. 23 and 24 were added to improve safety and were accepted by the NRC in a Safety Evaluation dated May 2, 1980. The licensee states that following this modification, the same technical specification requirements have been applied (although not specified in the Technical Specifications) to batteries nos. 23 and 24 as exist for batteries 21 and 22. The licensee states further that in the interest of consistency, batteries nos. 23 and 24 should be added to the Technical Specifications.

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The NRC staff has reviewed the May 2, 1980 Safety Evaluation and the Indian Point 2 one line diagrams. We find that the Technical Specifications should have been amended following the 1980 modifications to include batteries nos. 23 and 24 and their associated battery chargers and d.c. distribution systems. Although the licensee states that the appropriate requirements have been applied even in the absence of their inclusion in the Technical Specifications, we find that the modifications should be made to the Technical Specifications as requested by the licensee. We find that the proposed changes will add to the d.c. system reliability and are therefore acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 17, 1987

Principal Contributor:

A. Toalston