

MAY 23 1986

Docket No. 50-247

DISTRIBUTION

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✓ Docket File NRC PDR
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E. Jordan B. Grimes
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Dear Mr. O'Toole:

SUBJECT: AUTHORIZATION OF TECHNICAL SPECIFICATION CHANGE FOR INDIAN POINT
NUCLEAR GENERATING, UNIT NO. 2

This confirms our telephone authorization given on May 23, 1986 for a change in the Technical Specifications for the Indian Point Nuclear Generating Unit No. 2 as requested in your letter on May 23, 1986. Facility Operating License DPR-26 is hereby amended by changing the Technical Specifications exactly as requested in your May 23, 1986, letter, copy enclosed.

Copies of the license amendments and our Safety Evaluation for these Technical Specification changes will be sent to you when processed.

Steven A. Varga, Director
PWR Project Directorate No. 3
Division of PWR Licensing-A, NRR

cc: See next page

OFC	: PAD#3	MMU	: D/PAD#3	:	:	:
NAME	: MSlosson;pws:	SVarga	:	:	:	:
DATE	: 5/23/86	5/23/86	:	:	:	:

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Indian Point Nuclear Generating
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3.4

STEAM and POWER CONVERSION SYSTEM

Applicability

Applies to the operating status of the Steam and Power Conversion System.

Objective

To define conditions of the turbine cycle steam-relieving capacity. Auxiliary Feedwater System and City Water System operation is necessary to ensure the capability to remove decay heat from the core.

Specification

- A. The reactor shall not be heated above 350°F unless the following conditions are met:
- (1) A minimum ASME code approved steam-relieving capability of twenty (20) main steam valves shall be operable (except for testing).
 - (2) Three auxiliary feedwater pumps each capable of pumping a minimum of 400 gpm must be operable.
 - (3) A minimum of 360,000 gallons of water in the condensate storage tank and a backup supply from the city water supply.
 - (4) Required system piping, valves, and instrumentation directly associated with the above components operable.
 - (5) The main steam stop valves are operable and capable of closing in five seconds or less.
 - (6) The total iodine activity of I-131 and I-133 on the secondary side of the steam generator shall be less than or equal to 0.15 uCi/cc.
- B. Except as modified by 3.4.C below, if any of the conditions of 3.4.A above cannot be met within 72 hours*, the reactor shall be placed in the hot shutdown condition within the next 12 hours and subsequently cooled below 350°F using normal operating procedures.

* On a one time basis, the 72 hour action statement for main steam safety valves MS-49A and MS-45C may be extended for an additional 2 week period ending 4:45 p.m. on June 7, 1986 provided the remaining 18 safety valves are operable and the high flux trip setpoint is reduced to 78% of rated thermal power.