

September 30, 1985

Docket No. 50-247

Mr. John D. O'Toole  
Vice President  
Nuclear Engineering and Quality Assurance  
Consolidated Edison Company  
of New York, Inc.  
4 Irving Place  
New York, New York 10003

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Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. 97 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated June 18, 1985.

The amendment revises the Technical Specifications to limit overtime for critical shift job positions, changes the audit frequency of the Emergency Preparedness Program and Safeguards Contingency Plan, and clarifies the Quality Assurance Record retention requirements.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/JDNeighbors

Joseph D. Neighbors, Project Manager  
Operating Reactors Branch #1  
Division of Licensing

Enclosures:

1. Amendment No. 97 to DPR-26
2. Safety Evaluation

cc: w/enclosures  
See next page

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Mr. John D. O'Toole  
Consolidated Edison Company  
of New York, Inc.

Indian Point Nuclear Generating  
Station 1/2

cc:

Mayor, Village of Buchanan  
236 Tate Avenue  
Buchanan, New York 10511

Michael Blatt  
Director Regulatory Affairs  
Consolidated Edison Company  
of New York, Inc.  
Broadway and Bleakley Avenues  
Buchanan, New York, 10511

Robert L. Spring  
Nuclear Licensing Engineer  
Consolidated Edison Company  
of New York, Inc.  
4 Irving Place  
New York, New York 10003

Senior Resident Inspector  
U.S. Nuclear Regulatory Commission  
Post Office Box 38  
Buchanan, New York 10511

Brent L. Brandenburg  
Assistant General Counsel  
Consolidated Edison Company  
of New York, Inc.  
4 Irving Place - 1822  
New York, New York 10003

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Carl R. D'Alvia, Esquire  
Attorney for the Village of  
Buchanan, New York  
395 South Riverside Avenue  
Croton-on-Hudson, New York 10520

Mr. Jay Dunkleberger  
Office of Policy Analysis  
and Planning  
York State Energy Office  
Building 2, Empire State Plaza  
Albany, New York 12223

Ms. Ellyn Weiss  
Sheldon, Harmon and Weiss  
2001 S Street, NW, Suite 430  
Washington, D.C. 20006

Director, Technical Development  
Programs  
State of New York Energy Office  
Agency Building 2  
Empire State Plaza  
Albany, New York 12223

Mr. Peter Kokolakis, Director  
Nuclear Licensing  
New York Power Authority  
123 Main Street  
White Plains, New York 10601

Mr. Murray Selman  
Vice President, Nuclear Power  
Consolidated Edison Company of  
New York, Inc.  
Broadway and Bleakley Avenues  
Buchanan, New York 10511

Mr. Frank Matra  
Resident Construction Manager  
Consolidated Edison Company  
of New York, Inc.  
Broadway and Bleakley Avenues  
Buchanan, New York 10511

Ezra I. Bialik  
Assistant Attorney General  
Environmental Protection Bureau  
New York State Department of Law  
2 World Trade Center  
New York, New York 10047



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 97  
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated June 18, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

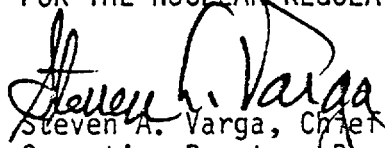
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 97, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 30, 1985

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 97 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

6-1

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6-10

6-22

Insert Pages

6-1

6-1a

6-10

6-22

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Vice President-Nuclear Power shall be responsible for overall facility activities as shown in Figure 6.2-2 and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The General Manager-Nuclear Power Generation shall be responsible for facility operations as shown in Figure 6.2-2 and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

FACILITY MANAGEMENT AND TECHNICAL SUPPORT

6.2.1 The organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

- 6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:
- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
  - b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
  - c. At least two licensed Operators shall be present in the control room during reactor startup, scheduled reactor shutdown, and during recovery from reactor trips.
  - d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
  - e. ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling. This individual shall have no other concurrent responsibilities during this operation.

- f. A Fire Brigade of at least five members shall be maintained on site at all times\*. This excludes four members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency. During periods of cold shutdown, the Fire Brigade will exclude two members of the minimum shift crew.
- g. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions (e.g., licensed Senior Operators, licensed Operators, health physicists, auxiliary operators, and key maintenance personnel).

The amount of overtime worked by unit staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).

\*Fire Brigade composition may be one member less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

## REVIEW (Continued)

- g. Reportable Events, as specified by 10 CFR 50.73.
- h. Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meeting minutes of the Station Nuclear Safety Committee.
- j. Environmental surveillance program pertaining to radiological matters.

## AUDITS

### 6.5.2.8

Audits of facility activities shall be performed under the cognizance of the NFSC. These audits shall encompass:

- a. The conformance of facility operation to all provisions contained within the Radiological Technical Specifications (Appendix A) and applicable license conditions at least once per 12 months.
- b. The conformance to all provisions contained within the Environmental Technical Specifications (Appendix B) pertaining to radiological matters and applicable license conditions at least once per 12 months.
- c. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- d. The results of all actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- e. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- f. The Facility Emergency Plan and implementing procedures at least once per 12 months.
- g. The Facility Security Plan and implementing procedures at least once per 12 months.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.



### Record Retention (continued)

- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual except as noted in 6.10.1.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the SNSC and the NFSC.
- l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.13.
- m. Record of analyses required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.

### 6.11 Radiation Protection Program

Procedure for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

### 6.12 High Radiation Area

6.12.1 As an acceptable alternate to the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

- a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1(a) above, and in addition locked doors shall be provided to prevent unauthorized entry to such areas and the keys shall be maintained under the administrative control of the Watch Supervisor on duty.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 97 TO FACILITY OPERATING LICENSE NO. DPR-26  
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.  
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2  
DOCKET NO. 50-247

Introduction

By letter dated June 18, 1985, the Consolidated Edison Company of New York requested an amendment to the Technical Specifications for Indian Point Nuclear Generating Unit No. 2 (IP-2). The amendment request proposes to incorporate into the IP-2 Technical Specifications overtime limits for critical shift job positions. The amendment request also proposes to revise the audit frequency of the Emergency Preparedness Program and Safeguards Contingency Plan and clarify the Quality Assurance (QA) record retaining requirements. The amendment application contained several other subjects which will be the subject of a separate review.

Discussion and Evaluation

NUREG-0737, "Clarification of TMI Action Plan Requirements" requires that overtime be limited for critical shift job positions. Generic Letter 82-12 provides the NRC Policy Statement on working hours. Generic Letter 82-16 requested licensees to revise their Technical Specifications to incorporate the guidance of Generic Letter 82-12. Consolidated Edison's proposed Technical Specification revision for limiting overtime is identical to the Standard Technical Specification included in Generic Letter 82-16. Therefore the proposed Technical Specification is acceptable.

Generic Letters 82-17 and 82-23 requested licensees to revise their Technical Specifications to require auditing of the Emergency Preparedness Program and the Safeguards Contingency Plan every 12 months, respectively. The revision to the Emergency Preparedness Plan auditing is required for the licensee to conform to 10 CFR 50.54(t) which requires an independent review of the Emergency Preparedness Plan at least every 12 months. The revision to the Safeguards Contingency Plan is required for the licensee to conform to the requirements of 10 CFR 73.40(d) which requires an independent review of the Safeguards Contingency Plan every 12 months. The current Technical Specifications require auditing at least every 24 months for both plans. The proposed changes are more stringent than the current Specifications and are required to meet the Code of Federal Regulations. Therefore the proposed Technical Specifications are acceptable.

The proposed change concerning QA retention requirements is required for clarification purposes only. The proposed revision is acceptable.

#### Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 30, 1985

#### Principal Contributor:

-M. Slosson