September 30, 1985

Docket No. 50-247

DISTRIBUTION

Docket File	NRC PDR
LPDR	ORB#1 Rdg
Gray File	HThompson
MSlosson	JDNeighbors
OELD	CParrish
LHarmon	EJordan
BGrimes	JPartlow
TBarnhart 4	WJones
MVirgilio	ACRS 10
CMiles	RDiggs
RBallard	

Mr. John D. O'Toole
Vice President
Nuclear Engineering and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. 99 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated July 31, 1985.

The amendment revises the Technical Specifications to permit a one-time extension of the surveillance interval limits for various systems and components. Specifically the Technical Specifications are modified to extend the 3.25 total time interval limit over three consecutive surveillance intervals to allow testing to be performed during the scheduled 1986 refueling/maintenance outage rather than requiring a special plant shutdown solely to perform these tests.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/JDNeighbors

Joseph D. Neighbors, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 99 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

ORB#1:DL	ORB#1:DL	ORB#1:DL	BC-ORB#1:DL	OELD
CParrish	MSlosson;ps	JDNeighbors	SVarga	
9/10/85	9/12/85	9/12/85	9/16/85	9/18/85

for MVirgilio
9/17/85

AD-OR:DL
GLainas
9/29/85

Mr. John D. O'Toole
Consolidated Edison Company
of New York, Inc.

cc:
Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, New York 10511

Michael Blatt
Director Regulatory Affairs
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenues
Buchanan, New York, 10511

Robert L. Spring
Nuclear Licensing Engineer
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 38
Buchanan, New York 10511

Brent L. Brandenburg
Assistant General Counsel
Consolidated Edison Company
of New York, Inc.
4 Irving Place - 1822
New York, New York 10003

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Carl R. D'Alvia, Esquire
Attorney for the Village of
Buchanan, New York
395 South Riverside Avenue
Croton-on-Hudson, New York 10520

Mr. Jay Dunkleberger
Office of Policy Analysis
and Planning
York State Energy Office
Building 2, Empire State Plaza
Albany, New York 12223

Ms. Ellyn Weiss
Sheldon, Harmon and Weiss
2001 S Street, NW, Suite 430
Washington, D.C. 20006

Director, Technical Development
Programs
State of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Mr. Peter Kokolakis, Director
Nuclear Licensing
New York Power Authority
123 Main Street
White Plains, New York 10601

Mr. Murray Selman
Vice President, Nuclear Power
Consolidated Edison Company of
of New York, Inc.
Broadway and Bleakley Avenues
Buchanan, New York 10511

Mr. Frank Matra
Resident Construction Manager
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenues
Buchanan, New York 10511

Ezra I. Bialik
Assistant Attorney General
Environmental Protection Bureau
New York State Department of Law
2 World Trade Center
New York, New York 10047

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISION COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 99
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated July 31, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 99 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 30, 1985

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Page	Insert Page
1-4	1-4

1.8 Quadrant Power Tilt Ratio

The quadrant power tilt ratio shall be the ratio of the maximum upper excore detector calibrated output to the average of the upper detector calibrated outputs, or the ratio of the maximum lower excore detector calibrated output to the average of the lower excore detector calibrated outputs, whichever is greater. With one excore detector inoperable, the remaining three detectors shall be used for computing the average.

1.9 Surveillance Intervals

Unless otherwise noted in an individual surveillance requirement, surveillance intervals shall be as specified in Table 1-1 with extensions as provided in 1.10 below. The extensions provided in 1.10 below also apply to surveillance intervals not listed in Table 1-1 unless the extensions are specifically not allowed.

1.10 Surveillance Interval Maximums

Each Surveillance Requirement shall be performed within the specified time interval with:

a. A maximum allowable extension not to exceed 25% of the surveillance interval, and

A total maximum combined interval time for any 3 consecutive surveillance intervals not to exceed 3.25 times the specified surveillance interval.*

1.11 PRESSURE BOUNDARY LEAKAGE

PRESSURE BOUNDARY LEAKAGE shall be leakage (except steam generator tube leakage) through a non-isolatable fault in a Reactor Coolant System component body, pipe wall or vessel wall.

1.12 IDENTIFIED LEAKAGE

IDENTIFIED LEAKAGE SHALL BE:

a. Reactor coolant system leakage into closed systems such as pump seal or valve packing leaks that are captured and conducted to a collecting tank, or

b. Reactor coolant system leakage through a steam generators to the secondary system, or

c. Reactor coolant system leakage through the RCS/RHR pressure isolation valves, or

* There shall be a one-time only exemption for surveillance requirements listed in Table 1 of the letter from John D. O'Toole to Steven A. Varga dated July 31, 1985. The 3.25 maximum combined interval may be extended to permit tests and calibrations to be performed prior to startup from the Cycle 7/8 refueling outage.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

Introduction

By letter dated July 31, 1985 Consolidated Edison Company of New York, Inc. (the licensee) requested a modification to the Indian Point Nuclear Generating Unit No. 2 (IP-2) Technical Specifications to permit a one-time extension of the surveillance interval limits for various systems and components shown in Table 1. The licensee requested that the Technical Specifications be modified to extend the 3.25 total time interval limit over three consecutive surveillance intervals to allow testing to be performed during the scheduled 1986 refueling/maintenance outage rather than requiring a special plant shutdown solely to perform these tests. The earliest surveillance test would have to be performed as early as October 1985. The 1986 refueling outage is scheduled to commence in January 1986.

Discussion and Evaluation

The IP-2 Technical Specification 1.10 states that each surveillance requirement shall be performed within the specified time interval with a maximum allowable extension not to exceed 25% of the surveillance interval and with a total maximum combined interval time for any 3 consecutive surveillance intervals not to exceed 3.25 times the specified surveillance interval. This is consistent with the Westinghouse Standard Technical Specifications. There are various components and systems for which the surveillance interval is given in the Technical Specifications as each refueling. A refueling interval is defined in the Technical Specifications as 18 months.

Generic Letter 83-27 dated July 6, 1983 entitled "Surveillance Intervals in Standard Technical Specifications" indicates that the 18 month surveillance interval is based on reactor operating experience and the recognition of reactors utilizing 18 month fuel cycles. The basis for the provision which allows any surveillance interval to be extended by 25% is to provide the necessary operational flexibility which may be required due to scheduling and operational performance considerations. Generic Letter 83-27 also indicates that one time changes may be granted for plant specific conditions where adequate justification is provided.

During the last three consecutive fuel cycles, IP-2 incurred two extended outages due to unplanned events and a long fuel cycle because of occasional reactor shutdowns and extended low power operation. As a result, for some equipment (Table 1) normally tested at an 18 months interval during refueling, the maximum combined interval of 3.25 times the specified surveillance will be reached before the next scheduled refueling outage. Without a one time extension to exceed the 3.25 criteria, IP-2 would be required to shutdown for an estimated five weeks to perform surveillance tests.

The licensee has indicated that the earliest that surveillance would be required is during October 1985. The next IP-2 refueling outage is scheduled for approximately the middle of January 1986. Therefore, the maximum extension time is approximately 3 months. This is an increase of 5% from the 3.25 criteria. However, Consolidated Edison has indicated that even with the extension, all surveillance tests for the equipment in Table 1 would be performed within the allowable existing permissible Technical Specification interval between any two tests, e.g., 18 months plus 25%.

The licensee has reviewed the results of previous surveillance tests and concluded that there is no reason to expect significant safety-related component failures during the extended surveillance interval.

The staff concludes that the quality of the components listed in Table 1 and its ability to perform will be maintained during the extension period to at least the equivalent of that level currently provided by the Technical Specifications for a maximum surveillance interval (i.e., 18 months plus 25%). Furthermore the staff concludes that extension of 5% is insignificant with regard to the surveillance interval and does not warrant an additional plant shutdown.

With regard to future calculations for the 3.25 criteria, this extension can be disregarded. However, it should be noted that although we are granting this one-time extension, the licensee should plan future surveillance in order that such extensions are not necessary.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria

for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 30, 1985

Principal Contributors:

M. Slosson
D. Neighbors

TABLE 1

SURVEILLANCE INFORMATION IN SUPPORT
OF PROPOSED TECHNICAL SPECIFICATION
CHANGE ON 3.25 MAXIMUM COMBINED INTERVAL

TEST NO	TEST DESCRIPTION	TECH SPEC REFERENCE
PC-R1A	REACTOR COOLANT LOOP RTD'S	TABLE 4.1-1 ITEM 4
PC-R1B	TAVG & DELTA T RTD'S	TABLE 4.1-1 ITEM 4
PC-R1C	WIDE RANGE COLD RTD'S	TABLE 4.1-1 ITEM 4
PC-R2	REACTOR COOLANT FLOW	TABLE 4.1-1 ITEM 5
PC-R5B	6.9 KV UNDER FREQUENCY RELAYS	TABLE 4.1-1 ITEM 8
PC-R9	RHR FLOW	TABLE 4.1-1 ITEM 13
PC-R13	CONTAINMENT LEVEL	TABLE 4.1-1 ITEM 17
PC-R14	CONTAINMENT PRESSURE	TABLE 4.1-1 ITEM 18
PC-R17A	ACCUMULATOR LEVEL	TABLE 4.1-1 ITEM 22
PC-R17B	ACCUMULATOR PRESSURE	TABLE 4.1-1 ITEM 22
PC-R18	STEAM LINE PRESSURE	TABLE 4.1-1 ITEM 23
PC-R19	TURBINE 1ST STAGE PRESSURE	TABLE 4.1-1 ITEM 24
PT-R2A	CONTAINMENT SUMP LEVEL	TABLE 4.1-1 ITEMS 21A & 21C
PT-R2B	RECIRC SUMP LEVEL	TABLE 4.1-1 ITEMS 21A & 21B
PT-R8	REFUELING INTERLOCKS	TABLE 4.1-3 ITEM 6
PT-R12	RHR SYSTEM HYDRO	SECTION 4.4.H.1
PT-R18	CONTAINMENT SPRAY ADDITIVE VALVE	SECTION 4.5.B.1
PT-R19	ACCUMULATOR CHECK & LOW HEAD INJ VALVE	SECTION 3.1.F.2.c
PT-R23	BORIC ACID ELECTRICAL HEAT TRACE	SECTIONS 3.2.B.5 & 3.3.A.1.b
PT-R34	SHOCK SUPPRESSORS (SNUBBER) FUNCTIONAL	SECTION 4.12.4
PT-R38	DEISEL GENERATOR BLDG WATER SYSTEM	SECTION 4.14.B.1.c & d
PT-R40	FCU'S WATER SPRAY & ALARMS	SECTION 4.14.B.1.c & d
PT-R44	FIRE PROTECTION SYSTEM VALVE CYCLING	SECTION 4.14.A.1.g.ii
PT-R47	FIRE DETECTION SYSTEM -SMOKE DETECTORS	SECTION 4.14.D.1.a.ii