Docket No. 50-247 February 28, 1985

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Mr. John D. O'Toole
Vice President
Nuclear Engineering and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. 93 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated February 28, 1984.

The amendment revises the Technical Specifications to incorporate the requirements of NUREG-0737 Item II.B.1, "Reactor Coolant System Vents." The Technical Specifications have been revised to ensure that the Indian Point Unit No. 2 Reactor Coolant Vent System is available to effectively vent noncondensible gases from the reactor coolant system without significantly increasing the probability of a Loss of Coolant Accident or challenge to containment integrity.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,
/s/JDNeighbors

Joseph D. Neighbors, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 93 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISION COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 93
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated February 28, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 93 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective on the date of issuance with implementation within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 28, 1985

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 93 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

| Remove Pages | Insert Pages |
|---|---|
| ii | ii |
| iii | iii |
| 3.1.2 | 3.1.2 |
| _____ | 3.16-1 |
| _____ | 3.16-2 |
| _____ | 4.20-1 |

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TABLE OF CONTENTS (Cont'd)

$\Delta$ T trip setpoint for three loop operation has been set in accordance with specification 2.3.1.B-4.

d. Reactor operation with one of the four loops out of service will be permitted for up to 24 hours. If the fourth loop can not be returned to service within 24 hours, the reactor will be put in a hot shutdown condition using normal procedures.

2. Steam Generator

Two steam generators shall be capable of performing their heat transfer function whenever the reactor is critical and the average coolant temperature is above 350°F.

3. Safety Valves

a. At least one pressurizer code safety valve shall be operable, or an opening greater than or equal to the size of one code safety valve flange shall be provided to allow for pressure relief, whenever the reactor head is on the vessel except for hydrostatically testing the RCS in accordance with Section XI of the ASME Boiler and Pressure Vessel Code.

b. All pressurizer code safety valves shall be operable whenever the reactor is critical.

c. The pressurizer code safety valve lift setting shall be set at 2485 psig with $\pm$1% allowance for error.

4. Power Operated Relief Valves (PORVs)/Block Valves

a. Whenever the reactor coolant system is above 350°F, the PORVs and their associated block valves shall be operable with the block valves either open or closed.

b. If a PORV becomes inoperable when above 350°F, its associated block valve shall be maintained in the closed position.

c. If a PORV block valve becomes inoperable when above 350°F, the block valve shall be closed and deenergized.

d. If the requirements of specifications 3.1.A.4.a, 3.1.A.4.b or 3.1.A.4.c above cannot be satisfied, compliance shall be established within one (1) hour, or the reactor shall be placed in the hot shutdown condition within the next six (6) hours and subsequently cooled below 350°F.

e. With regard to the use of the PORVs/Block Valves as a reactor coolant system vent, the requirements of specification 3.16 shall be adhered to.

## 3.16 REACTOR COOLANT SYSTEM VENTS

Applicability

Applies to the operability of the reactor coolant system vents.

Objective

To define those limiting conditions for operation that are necessary to ensure the ability to exhaust noncondensible gases from the primary coolant system.

Specification

A. Whenever the reactor coolant system is above 350°F the reactor vessel head vent, and at least one of the two pressurizer steam space vents shall be operable with associated valve positions as follows:

1. Reactor Vessel Head Vent:
   (a) HCV-3100 and HCV-3101 closed
   (b) Manual valve 500 open

2. Pressurizer Steam Space Vent No. 1:
   (a) PCV-455C closed
   (b) MOV-535 open or closed

3. Pressurizer Steam Space Vent No. 2:
   (a) PCV-456 closed
   (b) MOV-536 open or closed

In addition, the PORVs (PCV-455C and 456) may be operated as necessary beyond their use as a reactor coolant system vent in accordance with approved plant procedures.

B. With the reactor vessel head vent inoperable or both pressurizer steam space vents inoperable*, startup and/or power operation may continue provided the inoperable vent(s) is (are) maintained closed with power removed from the valve actuator of all the valves in the inoperable vent(s). Restore either the reactor vessel head vent or one of the two pressurizer steam space vents respectively to operable status to meet the requirements of 3.16.A within 30 days, or the reactor shall be placed in the hot shutdown condition within the next 6 hours, and in cold shutdown within the following 30 hours.

* The requirements of Specification 3.1.A.4 shall also be adhered to.

C. With all of the above reactor coolant system vents inoperable*, maintain the inoperable vents closed with power removed from the valve actuators of all the valves in the inoperable vents, restore one vent to operable status within 72 hours and apply the requirements of Specification 3.16.B. If this cannot be accomplished, the reactor shall be placed in the hot shutdown condition within the next 6 hours, and in cold shutdown within the following 30 hours.

**BASIS**

Reactor Coolant System Vents are provided to exhaust noncondensible gases from the primary coolant system. The operability of two reactor coolant system vents from the reactor vessel head and the pressurizer steam space ensures that capability exists to perform this function.

The valve redundancy of the reactor coolant system vents serves to minimize the probability of inadvertent or irreversible actuation while ensuring that a single failure of a vent valve power supply or control system does not prevent isolation of the vent path.

The function, capabilities, and testing requirements of the reactor coolant system vent systems are consistent with the requirements of Item II.B.1 of NUREG-0737, "Clarification of TMI Action Plan Requirements", November 1980.

**Reference**

FSAR-Section 4.2.10

* The requirements of Specification 3.1.A.4 shall also be adhered to.

## 4.20 REACTOR COOLANT SYSTEM VENTS

### Applicability

Applies to the periodic testing requirements for the reactor coolant system vents at refueling intervals.

### Objective

To verify the operability of the reactor coolant system vents and their ability to exhaust noncondensible gases from the primary system when required.

### Specification

A. Each reactor coolant system vent shall be demonstrated operable at refueling intervals by verifying flow through the reactor coolant system vents during cold shutdown.

### Basis

The requirement specified in 4.20.A establishes the surveillance test to be performed at refueling intervals to verify the operability of the reactor coolant system vents. This qualitative flow test will verify that the vents identified in specification 3.16.A. will be available to exhaust gases from the primary coolant system by demonstrating that no blockage exists in the vent system paths.

The periodic testing required by the ASME Code Section XI for each valve in the vents is conducted as specified in the Indian Point Unit No. 2 Inservice Inspection and Testing Program and is therefore not included in these specifications.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

# RELATED TO AMENDMENT NO. 93 TO FACILITY OPERATING LICENSE NO. DPR-26

# CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

# INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

# DOCKET NO. 50-247

## INTRODUCTION AND BACKGROUND

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements", which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of Technical Specifications for all of these items in Generic Letter 83-37. Generic Letter 83-37 was issued to all Pressurized Water Reactor (PWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facility's Technical Specifications to determine if they were consistent with the guidance provided in the Generic Letter, and

2. submit an application for a license amendment where deviations or absence of Technical Specifications were found.

By letter dated February 28, 1984, the Consolidated Edison Company of New York, Inc., (the licensee) responded to Generic Letter 83-37 by submitting a request for a change in the Technical Specifications for Indian Point Unit 2. This evaluation covers TMI Action Plan item II.B.1 - Reactor Coolant System Vents.

## EVALUATION

Our guidance for Reactor Coolant System (RCS) vents identified the need for at least one operable vent path at the reactor vessel head and the pressurizer steam space, for Westinghouse reactors. Generic Letter 83-37 also provided limiting conditions for operation and the surveillance requirements for the RCS vents. The licensee has proposed TSs that are consistent with our guidance. We find the proposed TSs to be acceptable.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 28, 1985

Principal Contributor:

C. Patel