

Docket No. 50-247

June 6, 1985

DISTRIBUTION

Docket File

Mr. John D. O'Toole
Vice President
Nuclear Engineering and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

L PDR	NRC PDR
HThompson	ORB#1 Rdg
MSlosson	CParrish
MHum	JDNeighbors
SECY	OELD
EJordan	LHarmon
JPartlow	BGrimes
WJones	TBarnhart 4
ACRS 10	EButcher
RBallard	RDiggs
	ORB#1 Gray

Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No.95 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated December 14, 1984.

The amendment revises the Technical Specifications to incorporate the requirements to perform augmented inservice inspection of the IP-2 reactor vessel during the second ten year inspection interval. The augmented inspection is required as a result of a flaw indication reported on the IP-2 reactor vessel during the cycle 6/7 refueling outage. In accordance with Section XI of the ASME Code, the amendment requires the inspection to be performed at a frequency of three times over the next ten years. Should any additional inspection demonstrate that the flaw is within the Section XI allowable for which no augmented inspection is required, the amendment allows the requirement for augmented inspection to become void.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/JDNeighbors

Joseph D. Neighbors, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No.95 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

*See previous white for concurrences

ORB#1:DL*	ORB#1:DL*	ORB#1:DL*
CParrish	MSlosson;ps	JDNeighbors
5/1 /58	5/1 /85	5/2 /85

BC-ORB#1:DL*
SVarga
5/3 /85

OELD	AD-OR:DL
<i>[Signature]</i>	GLainas
5/1/85	5/1/85
<i>per telecon 6/4/85</i>	

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GLainas
5/ /85

[Handwritten signatures and initials, including a large signature at the bottom right.]

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4/ /85

BC-ORB#1:DL
SVarga
4/ /85

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4/ /85

AD-OR:DL
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4/ /85

Mr. John D. O'Toole
Consolidated Edison Company
of New York, Inc.

Indian Point Nuclear Generating

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated December 14, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

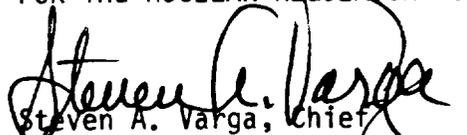
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 95, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 6, 1985

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 95 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
4.2-15	4.2-15
4.2-27	4.2.-27
-----	4.2-28

ITEM 6.6 (CATEGORY K-1) - Integrally - Welded Supports

There are no integrally - welded supports on the valves subject to this examination.

ITEM 6.7 (CATEGORY K-2) - Supports and Hangers

The supports and hangers of the valves subject to this examination shall be visually examined in accordance with Section XI of the code, as shown in Table 4.2-1.

G. Miscellaneous Inspections

ITEM 7.1 - Primary Pump Flywheels

The flywheels shall be visually examined at the first refueling. At each subsequent refueling, one different flywheel shall be examined by ultrasonic methods. The examination schedules are shown in Table 4.2-1.

ITEM 7.2 - Reactor Vessel Special Inspection

1. Interval of Inspection:

The reactor vessel shall be examined during the second ten year interval in the area of the vessel weld located approximately 236 inches below the reactor vessel flange at 345° azimuth. This area shall be reexamined during the three successive inspection periods as defined in accordance with IWB-2410 of the 1980 ASME Boiler & Pressure Vessel Code, Section XI, as modified below.

The examination schedule may revert to the original inspection schedule per IWB-2410 if:

- (i) The additional examinations reveal that the indications remain essentially unchanged over 3 successive inspections, or
- (ii) Any additional examination utilizing ultrasonic techniques per IWA-2232, or alternative techniques per IWA-2240, as supplemented by prior examination, demonstrate that the reflector meets the acceptance standards of IWB-3510. Such demonstration shall be submitted for NRC review and approval. Upon receipt of NRC concurrence, this special inspection requirement (item 7.2 in its entirety) shall become void.

2. Reporting Requirements:

The reactor vessel inservice inspection program shall be forwarded to NRC 180 days prior to plant shutdown during which the inspection is scheduled to be accomplished. Inspection results shall be forwarded for NRC review and approval 15 days prior to plant startup.

TABLE 4.2-1 (Sheet 11 of 11)

<u>Item No.</u>	<u>Examination Category</u>	<u>Components and Parts to be Examined</u>	<u>Method</u>	<u>Extent of Examination (Percent in 10 Year Interval)</u>	<u>Remarks</u>
6.4	G-1	Pressure-retaining bolting		Not applicable	
6.5	G-2	Pressure-retaining bolting	V	100%	Exception is taken for valves which are not accessible.
6.6	K-1	Integrally-welded supports		Not applicable	
6.7	K-2	Supports and hangers	V	100%	Exception is taken for supports and hangers which are not accessible.
MISCELLANEOUS INSPECTIONS					
7.1		Primary pump flywheel	V & UT	See Remarks	The flywheels shall be visually examined at the first refueling. At each subsequent refueling, one different flywheel shall be examined by ultrasonic methods.
7.2		Reactor Vessel Special Inspection Area	UT	See remarks	The reactor vessel shall be examined during the second ten year interval in the area of the vessel weld located approximately 236 inches below the reactor vessel flange at 345° azimuth. This area shall be reexamined during the three successive inspection periods as defined in accordance with NB-2410 of the 1980 ASME Boiler & Pressure Vessel Code, Section XI as modified below.

TABLE 4.2-1 (Sheet 11a of 11)

<u>Item No.</u>	<u>Examination Category</u>	<u>Components and Parts to be Examined</u>	<u>Method</u>	<u>Extent of Examination (Percent in 10 Year Interval)</u>	<u>Remarks</u>
					<p>The examination schedule may revert to the original inspection schedule per IWB-2410 if:</p> <p>(i) The additional examinations reveal that the indications remain essentially unchanged over 3 successive inspections; or</p> <p>(ii) Any additional examination utilizing ultrasonic techniques per IWA-2232, or alternative techniques per IWA-2240, as supplemented by prior examination, demonstrate that the reflector meets the acceptance standards of IWB-3510. Such demonstration shall be submitted for NRC review and approval. Upon receipt of NRC concurrence, this special inspection requirement shall become void.</p>



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 95 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

I. INTRODUCTION

During the 1984 refueling outage the licensee, Consolidated Edison Company, conducted a scheduled ten-year inservice inspection of the reactor pressure vessel. An ultrasonic flaw indication was detected on August 5, 1984 in the longitudinal weld at vessel location 345° azimuth in the lower shell course near the intersection of the circumferential girth weld between the lower shell course to middle shell course, 236 inches below the vessel flange. The staff met with Consolidated Edison representatives, reviewed the licensee's submittals related to the flaw indication, performed an independent fracture mechanics analysis and published a safety evaluation report, dated October 16, 1984. The staff has reasonable assurance that an indication with a through wall dimension of 1.2 inches and a length of 2.0 inches conservatively envelopes the actual flaw. The staff considers this flaw indication to be conditionally acceptable in accordance with subarticle IWB-3123 of Section XI of the ASME Code¹ and therefore requires augmented inservice inspection based on 10 CFR 50.55a(g)(4).

¹ Section XI, 1974 Edition including Addenda through Summer 1975.

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II. SUMMARY OF THE LICENSEE'S SUBMITTAL DATED DECEMBER 14, 1984

In Attachment A of a letter dated December 14, 1984 the licensee transmitted a proposed amendment to the Technical Specification. The licensee proposes to either reexamine the vessel weld located approximately 236 inches below the vessel flange at 345° azimuth during three successive inspection periods or perform sufficient examinations to demonstrate that the reflector meets the acceptance standards of subarticle IWB-3510 of Section XI. Upon concurrence from the staff that either of the inspection requirements has been met, the proposed special inspection provision of the Technical Specification shall become void.

III. CODE REQUIREMENT

Paragraph 10 CFR 50.55a(g)(4) requires that throughout the service life of a nuclear power facility, components which are classified as ASME Code Class 1 shall meet the requirements, except design and access provisions, set forth in the specific referenced editions of Section XI of the ASME Code to the extent practical within the limitations of design, geometry and materials of construction of the components.

The examination of ASME Code Class 1 components that either confirms the absence of, or reveals flaw indications that are not in excess of the standards listed in Article IWB-3000 of Section XI results in components

that are acceptable for service. Flaw indications that are in excess of the acceptance standards are not acceptable for service unless such flaws are removed or repaired. However, flaws in excess of the acceptance standards can be evaluated based on Section XI subarticle IWB-3123 "Conditional Acceptance" and determined to be acceptable for continued service provided that analyses confirm the structural adequacy of the component and the flaw indication is reexamined during prescribed scheduled inspections.

IV. STAFF EVALUATION

The staff has reviewed the proposed amendment to the Technical Specification related to the flaw indication in the reactor vessel that is defined in Attachment A of the licensee's letter dated December 14, 1984. The staff has determined that this proposed amendment is acceptable because the provisions of the proposed change conform with the augmented inservice inspection requirements of Section XI of the ASME Code for a flaw indication that is conditionally acceptable for service.

V. ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in

individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

VI. CONCLUSION

We have concluded, based on the considerations discussed above, that:

- (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner,
- and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 6, 1985

PRINCIPAL CONTRIBUTOR:

M. Hum