

Docket No. 50-247 December 7, 1984

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*Posted #2
Correction to
Amat. 90
to DPR-26*

Mr. John D. O'Toole
Vice President
Nuclear Engineering and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Dear Mr. O'Toole:

SUBJECT: RADIOLOGICAL EFFLUENT TECHNICAL SPECIFICATIONS (RETS) FOR INDIAN
POINT NUCLEAR GENERATING PLANT UNIT NO. 2

By letter dated June 6, 1984, Amendment No. 90 to Facility Operating License
DPR-26 for Indian Point Nuclear Generating Plant Unit 2 was issued. The
amendment was issued with three pages containing typographical and administra-
tive errors.

Page 4.10-8 was issued with the letter "C" displayed in the columns
"Sampling Frequency" and "Minimum Analysis Frequency" for the gaseous
release types "Waste Gas Storage Tank" and "Containment Purge". The letter
"C" does not correspond to any IP-2 frequency notations. The letter "C"
should be replaced by the letter "P".

The first paragraph on page 6-15 (a continuation from page 6-14) was
inadvertently omitted. This paragraph was not otherwise revised.

Paragraph 6.12.1.a on page 6-24 was issued with a typographical error. The
paragraph begins "Each High Radiation Area in which the intensity of
radiation is greater than 100 mrem/hr. but less than 100 mrem/hr. . ." It
should begin "Each High Radiation Area in which the intensity of radiation
is greater than 100 mrm/hr but less than 1,000 mrem/hr. . ."

A copy of the corrected pages are enclosed.

Sincerely,

/s/SVarga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

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JNeighbors
12/1/84

C-ORB#1:DL
SVarga
12/7/84



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 7, 1984

Docket No. 50-247

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Consolidated Edison Company
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4 Irving Place
New York, New York 10003

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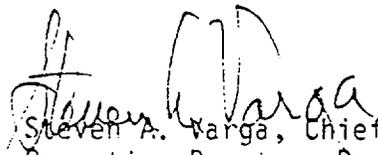
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cc w/enclosure:
See next page

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Indian Point Nuclear Generating Unit 2

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TABLE 4.10-3

RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

<u>Gaseous Releases Type</u>	<u>Sampling Frequency</u>	<u>Minimum Analysis Frequency</u>	<u>Type of Activity Analysis</u>	<u>Lower Limit of Detection (LLD)^a (uCi/ml)</u>
A. Waste Gas Storage Tank	C Each Tank Grab Sample	C Each Tank	Principal Gamma Emitters ^b	1x10 ⁻⁴
B. Containment PURGE	C Each PURGE Grab Sample	C Each PURGE	Principal Gamma Emitters ^b	1x10x ⁻⁴
C. Condenser Air Ejector	M ₁	M	Principal Gamma Emitters ^b	1x10 ⁻⁴
D. Plant Vent	M ^C Grab Sample	M ^C	Principal Gamma Emitters ^b	1x10 ⁻⁴
	Continuous ^f	W _e	H-3	1x10 ⁻⁶
	Continuous ^f	W _g	I-131	1x10 ⁻¹²
	Continuous ^f	Charcoal Sample		
		W	Principal Gamma Emitters ^b	1x10 ⁻¹¹
	Continuous ^f	Particulate Sample	(I-131, Others)	1x10 ⁻¹¹
		M	Gross Alpha	1x10 ⁻¹¹
		Composite	Particulate	
Continuous ^f	Sample			
	Q	Sr-89, Sr-90	1x10 ⁻¹¹	
Continuous ^f	Composite			
	Particulate Sample			
Continuous ^f	Noble Gas Monitor	Noble Gases Gross Beta or Gamma	1x10 ^{-6d}	

Superseded by
Correction Letter
12/7/84

Annual Radiological Environmental Operating Report 3

- 6.9.1.5 Routine Radiological Environmental Operating Reports covering the operation of the unit during the previous calendar year shall be submitted prior to May 1 of each year.

The Annual Radiological Environmental Operating Reports shall include summaries, interpretations, and statistical evaluation of the results of the radiological environmental surveillance activities for the report period, including a comparison with preoperational studies, with operational controls as appropriate, and with previous environmental surveillance reports, and an assessment of the observed impacts of the plant operation on the environment. The report shall also include the results of land use censuses required by Specification 4.11.B

The Annual Radiological Environmental Operating Reports shall include the results of analysis of all radiological environmental samples and of all environmental radiation measurements taken during the period pursuant to the locations specified in the Table and Figures in the ODCM, as well as summarized and tabulated results of these analyses and measurements as described in the ODCM. In the event that some individual results are not available for inclusion with the report, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted as soon as possible in a supplementary report.

The reports shall also include the following: a summary description of the radiological environmental monitoring program; at least two legible maps⁴ covering all sampling locations keyed to a table giving distances and directions from the centerline of one reactor; the results of licensee participation in the Interlaboratory Comparison Program, required by Specification 4.11.C; discussion and all deviations from the sampling schedule of Table 4.11-1; and discussion of all analyses in which the LLD required by Table 4.11-3 was not achievable.

³A single submittal may be made for a multiple unit station.

⁴One map shall cover stations near the site boundary; a second shall include more distant stations.

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12/7/84*

Record Retention (continued)

- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the SNSC and the NFSC.
- l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.13.
- m. Records of analyses required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.

6.11 Radiation Protection Program

Procedure for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 High Radiation Area

6.12.1 As an acceptable alternate to the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

- a. Each High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1(a) above, and in addition locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Watch Supervisor on duty.