

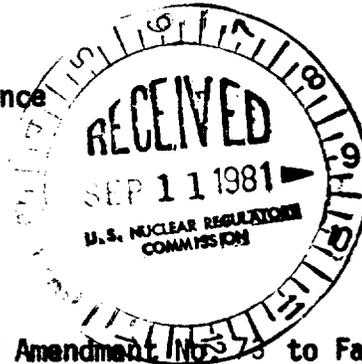
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SEP 04 1981

Docket No. 50-247

Mr. John D. O'Toole
 Vice President
 Nuclear Engineering and Quality Assurance
 Consolidated Edison Company
 of New York, Inc.
 4 Irving Place
 New York, New York 10003



Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. 73 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. This amendment consists of a temporary change to the Technical Specifications in response to your request transmitted by letter dated August 11, 1981, telecopied to us on August 11, 1981. This amendment was authorized by telephone on August 11, 1981 and confirmed by letter dated August 11, 1981.

The amendment temporarily extends the allowable time for a safety injection pump to be operable. This amendment was issued on an expedited basis to maintain the plant at a steady-state condition and avoid subjecting the plant to a shutdown transient shown by our evaluation to be unnecessary.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

John Hannon, Project Manager
 Operating Reactors Branch #1
 Division of Licensing

Enclosures:

1. Amendment No. 73 to DPR-26
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
 See next page

*Previous concurrence see next page

OFFICE	ORB#1:DL*	ORB#1:DL*	ORB#1:DL*	AD/OR:DL*	OELD*		
SURNAME	CParrish	JHannon:ds	Marga	TNovak			
DATE	8/ /81	8/ /81	8/ /81	8/ /81	8/ /81		

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 Chairman, ASLAB

Docket No. 50-247

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Dear Mr. O'Toole:

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The amendment temporarily extends the allowable time for a safety injection pump to be inoperable. This amendment was issued on an expedited basis to allow plant operation with one inoperable safety injection pump to continue for seven days.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

John Hannon, Project Manager
 Operating Reactors Branch #1
 Division of Licensing

Enclosures:

1. Amendment No. to DPR-26
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
 See next page

*FR NOTICE
 +
 AMENDMENT*

*no legal
 objection*

OFFICE	ORB#1:DL	ORB#1:DL	ORB#1:DL	AD/OP:DL	OELD		
SURNAME	CParrish	JHannon:ds	SVarga	TNovak	JIMBONE		
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The request for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated August 11, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

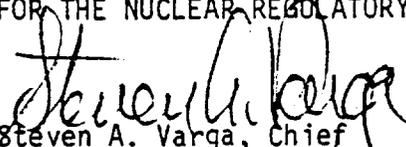
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of August 11, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 4, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Page

3.3-3

Insert Page

3.3-3

- a. One safety injection pump may be out of service, provided the pump is restored to operable status within 24 hours* and the remaining two pumps are demonstrated to be operable.
- b. One residual heat removal pump may be out of service, provided the pump is restored to operable status within 24 hours and the other residual heat removal pump is demonstrated to be operable.
- c. One residual heat removal exchanger may be out of service provided that it is restored to operable status within 48 hours.
- d. Any valve required for the functioning of the system during and following accident conditions may be inoperable provided that it is restored to operable status within 24 hours and all valves in the system that provide the duplicate function are demonstrated to be operable.
- e. One channel of heat tracing may be out of service for 48 hours.
- f. One refueling water storage tank low level alarm may be inoperable for up to 7 days provided the other low level alarm is operable.

B. Containment Cooling and Iodine Removal Systems

1. The reactor shall not be made critical unless the following conditions are met:
 - a. The spray additive tank contains not less than 4000 gallons of solution with a sodium hydroxide concentration of not less than 30% by weight.
 - b. The five fan cooler-charcoal filter units and the two spray pumps, with their associated valves and piping, are operable.
2. During power operation, the requirements of 3.3.B-1 may be modified to allow any one of the following components to be inoperable. If the system is not restored to meet

*One time only exception for safety injection pump 23, the 24 hour action statement for operability of safety injection pump 23 may be extended for a period of 7 days ending 6 AM August 17, 1981 provided safety injection pump 23 is returned to operable status as soon as maintenance is completed. The provisions of Section 1.10 of the technical specifications are not applicable for this one time change.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DUCKET NO. 50-247

Introduction

By telecopied letter dated August 11, 1981, the Consolidated Edison Company of New York (the licensee) requested a temporary change to the Technical Specifications appended to Facility Operating License No. DPR-26 for Indian Point, Unit No. 2. The change allows, one-time only, operation for up to seven days, ending 6:00 a.m. August 17, 1981, with one safety injection pump inoperable.

The request was generated after safety injection pump 23 malfunctioned while refilling an accumulator on August 10, 1981. Paragraph 3.3.A.2.a of the Technical Specifications allows continued power operation for 24 hours with one inoperable safety injection pump. The requested change added a footnote to this paragraph to allow a one-time extension to 7 days.

Evaluation

The licensee included a safety evaluation of the requested change which we have reviewed. We agree that it is highly unlikely that a need for the high head safety injection (HHSI) pumps would arise within the seven-day period.

However, in the unlikely event that these HHSI pumps are required, the licensee has evaluated the consequences of losing two pumps (one pump inoperable, the other lost as a result of a single failure).

The loss of the HHSI pump has an impact on the small break Loss-of-Coolant Accident (LOCA) analysis. The existing analysis predicts a peak clad temperature (PCT) of 1380°F. This analysis assumed two of three HHSI pumps available and no credit for the charging pumps. With one HHSI lost as a result of the worst single failure, loss of a diesel, core cooling capability is reduced. Sensitivity studies presented by the licensee indicate that the PCT could increase approximately 700°F.

In addition, the licensee stated that additional margin exists because a recently approved Westinghouse model was not used but would reduce PCT by about 200°F, and because the decay heat function used and required by Appendix K (ANS + 20%) has been demonstrated to be conservative. A better estimate of the decay heat function would result in about a 250°F reduction in PCT.

As a compensatory measure for the seven-day period, the operators at Indian Point Unit 2 have been instructed to restore the charging pumps to service following safety injection. Having flow from two of three charging pumps would further mitigate the consequences of a small break LOCA.

We therefore conclude that, in the unlikely event of a LOCA during this seven-day period, the PCT limit of 10 CFR 50.46 (2200°F) and the February 11, 1980 Confirmatory Order (2000°F) will not be exceeded.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: September 4, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-247CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 73 to Facility Operating License No. DPR-26, issued to the Consolidated Edison Company of New York, Inc. (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 2 (the facility) located in Buchanan, Westchester County, New York. The amendment was effective August 11, 1981.

The amendment was authorized by telephone on August 11, 1981 and was confirmed by letter dated August 11, 1981. The amendment changes the Technical Specifications, one-time only, to allow continued power operation up to seven days, ending 6:00 a.m. August 17, 1981, with one safety injection pump inoperable. The amendment was authorized on an expedited basis to maintain the plant at a steady-state condition and avoid a shutdown transient shown by our evaluation to be unnecessary but required by Technical Specifications unless amended.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which

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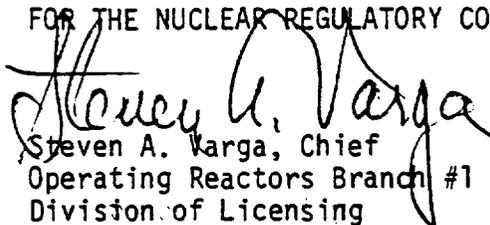
are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the request for amendment dated August 11, 1981, (2) the Commission's letter to the licensee dated August 11, 1981, (3) Amendment No. 73 to License No. DPR-26, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 4th day of September, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing