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Docket No. 50-247

Correction ltr to Amdt. 74

Mr. John D. O'Toole, Vice President
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Dear Mr. O'Toole:

On December 10, 1981, the Commission issued Amendment No. 74 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment modified your Technical Specifications to account for the effects that the degraded grid voltage may have on plant operations.

Through administrative oversight, certain parts of Table 4.1-1 were incorrectly modified by the amendment. Enclosed you will find the corrected Technical Specification pages for Table 4.1-1. Also enclosed is the first page of our Safety Evaluation for Amendment No. 74 which your staff indicated was not included with our package. We apologize for any inconvenience this administrative oversight may have caused.

Sincerely,

ORIGINAL SIGNED
John Thoma, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
As stated

cc w/enclosures:
See next page

JOT

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TABLE 4.1-1 (CONTINUED)

Channel Description	Check	Calibrate	Test	Remarks
22. Accumulator Level and Pressure	S	R	N.A.	
23. Steam Line Pressure	S	R	M	
24. Turbine First Stage Pressure	S	R	M	
25. Logic Channel Testing	N.A.	N.A.	M	
26. Turbine Overspeed Protection Trip Channel (Electrical)	N.A.	R	M	
27. Control Room Ventilation	N.A.	N.A.	R	Check damper operation for accident mode with isolation signal
28. Control Rod Protection (for use with LOPAR fuel)	N.A.	R	*	
29. Loss of Power				
a. 480v Emergency Bus Undervoltage (Loss of Voltage)	N.A.	R	R	
b. 480v Emergency Bus Undervoltage (Degraded Voltage)	N.A.	R	R	
c. 480v Emergency Bus Undervoltage (Alarm)	N.A.	R	M	
30. Auxiliary Feedwater:				
a. Steam Generator Water Level (Low-Low)	S	R	R	

* Within 31 days prior to entering a condition in which the Control Rod Protection System is required to be operable unless the reactor trip breakers are manually opened during RCS cooldown prior to T_{cold} decreasing below 350°F and the breakers are maintained open during RCS cooldown when T_{cold} is less than 350°F.

TABLE 4.1-1 (Continued)

CHANNEL DESCRIPTION	CHECK	CALIBRATE	TEST
b. Station Blackout (Undervoltage)	N.A.	R	R
c. Trip of Main Feedwater Pumps	N.A.	N.A.	R
31. Reactor Coolant System Subcooling Margin Monitor	M	R	N.A.
32. PORV Position Indicator (Limit Switch)	M	R	R
33. PORV Block Valve Position Indicator (Limit Switch)	M*	R	R
34. Safety Valve Position Indicator (Acoustic Monitor)	M	R	R
35. Auxiliary Feedwater Flow Rate	M	R	R
36. PORV Actuation/Reclosure Setpoints	N.A.	R	N.A.

* Except when valve operator is deenergized in accordance with specification 3.1.A.4.c

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

Introduction and Summary

A request for certain information on the effects that degraded grid voltage may have on plant operations was sent to Consolidated Edison Company (Con-Ed) by the NRC on August 12, 1976. General Design Criterion 17 (GDC 17), "Electric Power Systems," of Appendix A, "General Design Criteria for Nuclear Power Plants," of 10 CFR Part 50 requires that the safety function of each a.c. system shall be to provide sufficient capacity and capability to assure that: (a) specified acceptable fuel design limits and the design conditions for the reactor coolant pressure boundry are not exceeded as a result of anticipated operational occurrences; and (b) the core is cooled and containment integrity and other vital functions are maintained during any of the postulated accidents. GDC 17 further requires that provisions be included to minimize the probability of losing electric power from any one of the remaining supplies as a result of or coincident with the loss of power generated by the nuclear power unit, the loss of power from the transmission network, or the loss of power from the onsite electric power supplies. A sustained degradation of the offsite power system's voltage could result in the loss of capability of the redundant safety loads, their control circuitry, and the associated electrical components required for performing safety functions. Criteria, staff positions, and proposed technical specifications on degraded grid protection were sent to the licensee by a generic letter dated June 2, 1977. Conformance to the standard technical specifications should provide adequate protection for the degraded grid voltage condition.

Con-Ed's responses were dated Spetember 24, 1976; March 31, 1977; August 29, 1977 (two letters); October 16, 1979; April 28, 1980; August 1, 1980; and April 27, 1981. During our review of their proposed technical specifictions, we found that certain changes were necessary. The licensee agreed to these changes and they have been incorporated. The detailed reviews and technical evaluations of Con-Ed's proposed plant modifications and technical specifications changes were performed by EG&G under contract to the NRC with general supervision provided by the staff. EG&G's Technical Evaluation Report is provided as Attachment 1 to this Safety Evaluation Report. We have reviewed EG&G's Technical Evaluation