

APR 28 1982

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CParrish
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RDiggs
LSchneider
ASLAB
DBrinkman
Gray

Docket No. 50-247

Mr. John D. O'Toole, Vice President
Nuclear Engineering and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. 76 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your request transmitted by telecopy dated March 24, 1982, as confirmed by letter dated March 26, 1982. This amendment was authorized by telephone on March 24, 1982 and was confirmed by letter dated March 25, 1982.

The amendment on a one-time basis allows an additional fifteen days extension to the maximum time between tests specified in Table 4.1-3 for the turbine stop and control valve test;

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

John Hannon, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

- 1. Amendment No. 76 to DPR-26
- 2. Safety Evaluation
- 3. Notice of Issuance

cc w/enclosures:
See next page



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Previous concurrences see next page

OFFICE	ORB #1 : DL	AD/OR : DL	OELD				
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Sincerely,

John N. Hannon, Project Manager
 Operating Reactors Branch #1
 Division of Licensing

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OFFICE	ORB#1:DL CParrish	ORB#1:DL JThoma ds	ORB#1:DL JHannon	ORB#1:DL SVarga	AD/OR:DL TNovak	OELD JHannon
SURNAME	CParrish	JThoma ds	JHannon	SVarga	TNovak	JHannon
DATE	04/9/82	04/9/82	04/9/82	04/9/82	04/12/82	04/21/82

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Mayor, Village of Buchanan
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 76
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The requests for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated March 24, 1982 and March 26, 1982, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the requests, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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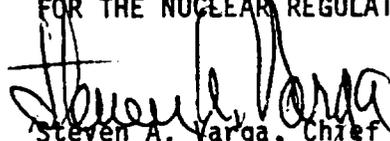
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 76, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective March 24, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 28, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 76

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Page

Table 4.1-3

Insert Page

Table 4.1-3

TABLE 4.1-3

FREQUENCIES FOR EQUIPMENT TESTS

	<u>Check</u>	<u>Frequency</u>	<u>Maximum Time Between Tests</u>
1. Control Rods	Rod drop times of all control rods	Each refueling shutdown	**
2. Control Rods	Partial movement of all control rods	Every 2 weeks during reactor critical operations	20 days
3. Pressurizer Safety Valves	Set point	Each refueling shutdown	**
4. Main Steam Safety Valves	Set point	Each refueling shutdown	**
5. Containment Isolation System	Automatic Actuation	Each refueling shutdown	**
6. Refueling System Interlocks	Functioning	Each refueling shutdown prior to refueling operation	NA*
7. <u>DELETED</u>			
8. Diesel Fuel Supply	Fuel Inventory	Weekly	10 days
9. Turbine Steam Stop, Control Valves	Closure	Monthly****	45 days**** +
10. Cable Tunnel Ventilation Fans	Functioning	Monthly	45 days
11. Control Room and Fuel Handling Building Filtration System	Charcoal Filter Pressure Drop Test < 5 inches of water visual inspection Freon - 112 (or equivalent) test ≥ 99.5% at ambient conditions	Each refueling shutdown prior to refueling operation***	**

Amendment No. 76

†One-time only exception, the 45-day maximum time between tests may be extended to 60 days ending at 11:59 P.M. on April 8, 1982.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 76 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

Introduction

The Indian Point Unit No. 2 Technical Specifications require performance of a main turbine steam stop and control valve closure test with a maximum allowed time between tests of 45 days. This test was to be performed no later than March 24, 1982. By application dated March 24, 1982, the licensee requested an amendment to defer this test for fifteen days until April 8, 1982.

There are a total of eight valves involved in the main turbine steam stop and control valve test. Six of these valves had been successfully tested before the test mechanism became inoperable preventing testing of the other two valves. The inoperability of the test mechanism does not prevent proper operation of the valves involved, but simply precludes testing them. A plant shutdown is necessary in order to repair the test mechanism. The licensee has requested a fifteen day one-time only extension to allow operation while awaiting delivery of necessary replacement parts to repair the test mechanism.

Evaluation

The basis for conducting a periodic main turbine steam stop and control valve closure test is to insure operability of these valves in a turbine overspeed condition. If these valves were to remain in a single position for an extended period of time, corrosive product buildup on the stem may prevent proper operation of the valve. Periodic cycling of these valves removes any corrosive product buildup and allows early detection of potential problems.

The licensee states that these valves have demonstrated consistently reliable performance throughout their history of operation. All valves were successfully tested 45 days earlier. Therefore, any corrosive product buildup on these valves should be fairly minimal. Six of the eight valves have been successfully tested now so basically the extension only applies to two valves. Furthermore, the licensee states that the inoperable test mechanism does not affect proper operation of the valves themselves. The fifteen day extension is to allow time to obtain vital spare parts for the test mechanism. We, therefore, concur that there is sufficient justification to delay this test on a one-time only basis for fifteen days.

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Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 28, 1982

Principal Contributor:
John Thoma

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-247CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 76 to Facility Operating License No. DPR-26, issued to the Consolidated Edison Company of New York, Inc. (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 2 (the facility) located in Buchanan, Westchester County, New York. The amendment was effective March 24, 1982.

The amendment on a one-time only basis allows an additional fifteen day extension to the maximum time between tests specified in Table 4.1-3 for the turbine stop and control valve closure test. The amendment was authorized on an expedited basis to maintain the plant at a steady-state condition and avoid a shutdown transient shown by our evaluation to be unnecessary but required by Technical Specifications unless amended.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

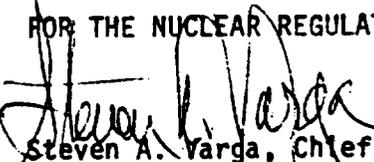
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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the requests for amendment dated March 24, 1982 and March 26, 1982, (2) the Commission's letter dated March 25, 1982, (3) Amendment No. 76 to License No. DPR-26, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2), (3), and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 28th day of April, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing