



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23T85
ATLANTA, GEORGIA 30303-8931

December 8, 2000

Virginia Electric and Power Company
ATTN: Mr. David A. Christian
Senior Vice President and
Chief Nuclear Officer
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060

SUBJECT: MEETING SUMMARY - REACTOR OVERSIGHT PROGRAM REGIONAL
PUBLIC MEETING

Dear Mr. Christian:

This refers to the Reactor Oversight Program meeting conducted on November 16, 2000, at the Sheraton Gateway Hotel in College Park, Georgia. A copy of the meeting agenda, list of attendees, presentation handouts, NRC Inspection Manual Chapter 0305 Action Matrix, and summary of meeting feedback issues are enclosed.

It is our opinion that this meeting was beneficial, in that it provided an opportunity for NRC, licensees, NEI, and the public to discuss the progress, challenges, and successes related to the initial implementation of the Reactor Oversight Program for Region II facilities.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosures will be placed in the NRC Public Document Room.

Should you have any questions concerning this meeting, please contact Mr. Robert Carroll at (404) 562-4511.

Sincerely,

//RA//

Victor M. McCree, Deputy Director
Division of Reactor Projects

Docket Nos. 50-338, 50-339, 50-280, 50-281
License Nos. NPF-4, NPF-7, DPR-32, DPR-37

Enclosures: As stated

cc w/encls: (See page 2)

cc w/encls: (Cont'd)
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Nuclear Licensing and
Operations Support
Virginia Electric and Power Company
Electronic Mail Distribution

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DATE	10/23/00	1/ /2001	1/ /2001	1/ /2001	1/ /2001	1/ /2001	1/ /2001
E-MAIL COPY?	YES NO						

Proposed Meeting Agenda
(Reactor Oversight Program Regional Public Meeting)

Date: November 16, 2000

Time: 8:00 a.m to 5:00 p.m.

Place: Sheraton Gateway Hotel
1900 Sullivan Road
College Park, GA 30337
(770) 994-2452

Agenda:

- | | |
|------------|--|
| 7:30 a.m. | Meeting Registration (for those not able to pre-register) |
| 8:00 a.m. | Introductory Remarks - Regional Administrator |
| 8:30 a.m. | Panel and General Discussion/Q&A
(Significance Determination Process) |
| 10:00 a.m. | Break |
| 10:30 a.m. | Panel and General Discussion/Q&A
(Inspections) |
| 12:00 noon | Lunch |
| 1:00 p.m. | Panel and General Discussion/Q&A
(Performance Indicators) |
| 2:30 p.m. | Break |
| 3:00 p.m. | Panel and General Discussion/Q&A
(Assessment, Enforcement) |
| 4:30 p.m. | Closing Remarks |

LIST OF ATTENDEES

Carolina Power and Light Company

McCartney, Eric - Supervisor, Licensing and Regulatory Programs, Harris
Nicely, Kenneth - Senior Engineer/Licensing, Brunswick
Statile, Donald - Principal Engineer/Licensing and Reg. Programs, Robinson

Dominion Generation

Olson, Donald - Coordinator, External Relations, Nuclear Licensing and Operations Support

Duke Energy Corporation

Cross, Randy - Technical Specialist/Nuclear Generation, Duke Power
Gilbert, Gary - Regulatory Compliance Manager, Catawba
Nicholson, Larry - Regulatory Compliance Manager, Oconee
Sweigart, Richard - Safety Assurance Manager, Catawba

Florida Power Corporation

Herrin, Dennis - Principal Nuclear Licensing Engineer, Crystal River 3

Florida Power and Light Company

Casto, Greg - Protection Services, St. Lucie
Franzone, Steve - Licensing Manager, Turkey Point

INPO

Felgate, George - Vice President and Director, Analysis Division
Johnson, Steve - Staff Assistant for Government Relations
Welsh, Gary - Manager, Performance Analysis Department

MEAG Power

Lindall, Cole - Assistant Manager, Generation

NEI

Floyd, Steve - Senior Director Regulatory Reform
Houghton, Tom - Senior Project Manager, Regulatory Reform
Marion, Alex - Director, Licensing

South Carolina Electric & Gas Company

Estes, Tyndall - PSA Team Leader/NL&OE, V. C. Summer
Reese, Susan - Licensing Specialist, V. C. Summer

Southern Nuclear Operating Company, Inc.

Cato, Andy - Vogtle Site PI Coordinator
Davis, Tommy - Vogtle Project PI Coordinator
Hill, Richard - General Manager, Nuclear Support
Kale, Jack - Engineering/Support
Nesbitt, Charles - Assistant General Manager, Plant Support
Warren, Wade - Corporate PI Coordinator

Tennessee Valley Authority

Abney, Tim - Manager, Licensing & Industry Affairs, Browns Ferry
Burzynski, Mark - Manager, Nuclear Licensing

Troutman Sanders

Kim, Judy

Virginia Electric and Power Company

Crossman, James - Supervising Licensing, Dominion-Virginia Power, North Anna

Nuclear Regulatory Commission

Berkow, Herbert - Project Director, Project Director II, Office of Nuclear Reactor Regulation (NRR)
Bartley, Jonathan - Senior Resident Inspector (SRI), Watts Bar, Region II (RII)
Boland, Anne - Director, Office of Enforcement, RII
Bonser, Brian - Chief, Reactor Projects Branch 4, Division of Reactor Projects (DRP), RII
Cahill, Stephen - Chief, Reactor Projects Branch 2, DRP, RII
Carroll, Robert - Project Engineer, Reactor Projects Branch 1, DRP
Casto, Charles - Director, Division of Reactor Safety (DRS), RII
Coe, Doug - Section Chief, Reactor Inspection Section, Division of Inspection Program Management (DIPM), NRR
Easlick, Theodore - SRI, Brunswick, RII
Fredrickson, Paul - Chief, Reactor Projects Branch 6, DRP, RII
Haag, Robert - Chief, Reactor Projects Branch 5, DRP, RII
Hickman, Don - Sr. Reactor Operations Engineer, DIPM, NRR
Madison, Alan - Sr. Reactor Operations Engineer, DIPM, NRR
Maley, Michael - Reactor Operations Engineer, DIPM, NRR
McCree, Victor - Deputy Director, DRP, RII
Ogle, Charles - Chief, Engineering Branch, DRS
Pascarelli, Robert - Reactor Operations Engineer, DIPM, NRR
Plisco, Loren - Director, Division of Reactor Projects, RII
Reyes, Luis - Regional Administrator, RII
Roberts, Darrell - SRI, Catawba, RII
Rogers, Walter - Sr. Reactor Analyst, DRS, RII
Rosenberg, Stacey - Region II Coordinator, Executive Director for Operations
Spector, August - Communication Task Lead, DIPM, NRR
Stein, Steve - Sr. Reactor Operations Engineer, DIPM, NRR
Stewart, Scott - SRI, Crystal River, RII
Wert, Leonard - Chief, Reactor Projects Branch 3, DRP, RII
Widmann, Malcolm - SRI, Summer, RII
Zeiler, John - SRI, Vogtle, RII

PRESENTATION HANDOUTS

Enclosure 3



NOVEMBER 16, 2000, REACTOR OVERSIGHT PROGRAM REGIONAL PUBLIC MEETING (PANEL PRESENTATION SLIDES)

**Performance Indicators
Inspections
Significance Determination Process
Assessment**



PERFORMANCE INDICATORS

MEANINGFUL INDICATOR THRESHOLDS:

- SCRAMS - RED >25/YEAR
- RCS LEAKAGE - >5GPM



PERFORMANCE INDICATORS

PIS DRIVING LICENSEE PERFORMANCE:

- DELAYING NEEDED REPAIRS FOR GREATER THAN 72 HOURS
- CHANGING PROCEDURES TO REQUIRE POWER REDUCTION OF < 20% IN RESPONSE TO OFF NORMAL CONDITIONS
- UNREASONABLE LEVEL OF MANAGEMENT ATTENTION TO PLAN AND SCHEDULE EQUIPMENT OUT OF SERVICE TIME
- CHANGING SURVEILLANCE PERIODS TO AVOID T/2 HITS



PERFORMANCE INDICATORS

FAQS BECOMING TOO CUMBERSOME:

- REVISION TO NEI 99-02



PERFORMANCE INDICATORS

DRILL EXERCISE PERFORMANCE:

- CLASSIFICATIONS PERFORMED IN THE SIMULATOR DURING REQUALIFICATION MASK POOR PERFORMANCE DURING DRILL/ACTUAL EVENT CLASSIFICATIONS



PERFORMANCE INDICATORS

NEW PIS:

- SBO EDG
- HUMAN PERFORMANCE



INSPECTIONS

INSPECTION SCHEDULING:

- NOTIFICATION TIMELINESS
- ADHERENCE TO SCHEDULE ISSUED IN LETTER
- ADHERENCE TO NUMBER OF INSPECTORS AND PROCEDURE SPECIFIED IN LETTER
- FLEXIBILITY OF SCHEDULING
- IMPACT ON REFUELING OUTAGES OR CONFLICTS WITH INPO EVALUATIONS



INSPECTIONS

INSPECTION PROCEDURES:

- CLARITY
- PREDICTABLE AND REPEATABLE
- APPROPRIATENESS OF FOCUS
- PREPARATIONS FOR INSPECTIONS



INSPECTIONS

EXITS, ENTRANCES, AND BRIEFINGS:

- COMMUNICATIONS OF FINDINGS
- APPROPRIATE COMMUNICATIONS OF INSPECTION ACTIVITY
- OVERALL USEFULNESS



INSPECTIONS

INSPECTION REPORTS:

- SUFFICIENT INFORMATION INCLUDED
- VIOLATIONS APPROPRIATELY COMMUNICATED
- FOCUS ON RISK
- USEFULNESS OF PRODUCT
- QUARTERLY ISSUANCE
- INTEGRATED REPORTS



INSPECTIONS

TEAM (OR MAJOR) AND SUPPLEMENTAL INSPECTIONS (PI&R, SSDI, FP, 95001, 95002):

- CONDUCTED IN ACCORDANCE WITH INSPECTION PROCEDURES
- PREPARATIONS (INCLUDING PRE-VISITS)
- OVERALL EFFICIENCY
- APPROPRIATE DEPTH AND FOCUS
- REACTIVE INSPECTIONS (AIT)



INSPECTIONS

INSPECTION FINDINGS:

- CONSISTENT WITH INSPECTION PROCEDURES AND OTHER GUIDANCE
- WITHIN SCOPE OF INSPECTION
- RISK SIGNIFICANT
- DIFFERENCES CLEAR (MINOR, NCV, GREEN, NO COLOR)



SIGNIFICANCE DETERMINATION PROCESS (SDP)

REACTOR SAFETY SDP:

- THE PROCESS FOCUSES ON RISK SIGNIFICANT ITEMS IN A REPEATABLE AND PREDICTABLE MANNER. CLEAR BASIS PROVIDED FOR SIGNIFICANCE DETERMINATIONS.
- INCREASED AMOUNTS OF RISK INFORMATION IS BEING COMMUNICATED BETWEEN THE NRC (PARTICULARLY THE SRAS) AND LICENSEE'S PRA STAFF TO SUPPORT SDP EVALUATIONS.
- SITE SPECIFIC PHASE 2 WORKSHEETS HAVE NEEDED REVISIONS TO ACCURATELY REFLECT PLANT CONFIGURATIONS.



SIGNIFICANCE DETERMINATION PROCESS (SDP)

REACTOR SAFETY SDP (CONT'D):

- THE IMPACT (IN TERMS OF BENEFITS AND DISTRACTIONS) ON LICENSEES AS THEY PERFORM RISK ASSESSMENTS IN RESPONSE TO NRC'S SDP REVIEWS.



SIGNIFICANCE DETERMINATION PROCESS (SDP)

FIRE PROTECTION SDP:

- COMPLEXITY OF PROCESS REQUIRES A HIGHER LEVEL OF EXPERTISE AND MORE TIME THAN OTHER SDPS.
- FIRE PROTECTION SDP HAS BETTER FOCUSED NRC'S INSPECTIONS AND REVIEWS ON RISK SIGNIFICANT AREAS/ ISSUES.
- EFFECTS FROM RECENT CHANGES IN THE NRC'S REVIEW OF "HOT SHORTS".
- CRITERIA USED IN EVALUATING FIRE BRIGADE PERFORMANCE.



SIGNIFICANCE DETERMINATION PROCESS (SDP)

PHYSICAL PROTECTION SDP:

- CHANGES IN PHYSICAL PROTECTION SDP AND IMPACT ON NRC/LICENSEE'S.
- USE OF SDP TO EVALUATE FORCE-ON-FORCE EXERCISE ISSUES HAS BEEN CHALLENGING.

COMMENTS ON OTHER SDPs, such as SHUTDOWN SAFETY, OPERATOR LICENSING, OCCUPATIONAL/ PUBLIC RADIATION SAFETY.



SIGNIFICANCE DETERMINATION PROCESS (SDP)

GENERAL COMMENTS ON SDPS:

- GROUP 1, 2 AND 3 QUESTIONS IN MANUAL CHAPTER 0609 HAVE A DEGREE OF SUBJECTIVITY. HAVE EXAMPLES OF INCONSISTENCY BEEN NOTED IN THE APPLICATION OF THESE QUESTIONS?
- FINDINGS WITH “NO-COLOR” DESIGNATION HAVE BEEN ISSUED IN INSPECTION REPORTS. DO QUESTIONS EXIST ON THESE TYPES OF FINDINGS AND WHY THE SDP DOES NOT CHARACTERIZE THEIR RISK SIGNIFICANCE?



SIGNIFICANCE DETERMINATION PROCESS (SDP)

GENERAL COMMENTS ON SDPS (CONT'D):

- HAVE LICENSEE RISK INSIGHTS BEEN ADEQUATELY CONSIDERED BY THE STAFF DURING THE SDP EVALUATIONS?
- DOES EXTENSIVE AND LENGTHY ANALYSIS TO CHARACTERIZE AN ISSUE AS GREEN DETRACT FROM CORRECTIVE ACTION FOCUS?
- HOW DOES THE RELATIVE LACK OF AVAILABLE PLANT-SPECIFIC EXTERNAL EVENT RISK ANALYSES AFFECT OUR CONFIDENCE IN THE REACTOR SAFETY SDP RESULTS?



ASSESSMENT

ASSESSMENT:

- APPLICATION OF THE ACTION MATRIX
- ASSESSMENTS PREDICTABLE/REPEATABLE
- ASSESSMENT LETTERS (END/MID-CYCLE) USEFUL
- MATRIX ACTIONS APPROPRIATE
- CONCERNS WITH “AGING” OF FINDINGS



ASSESSMENT

ASSESSMENT (CONT'D):

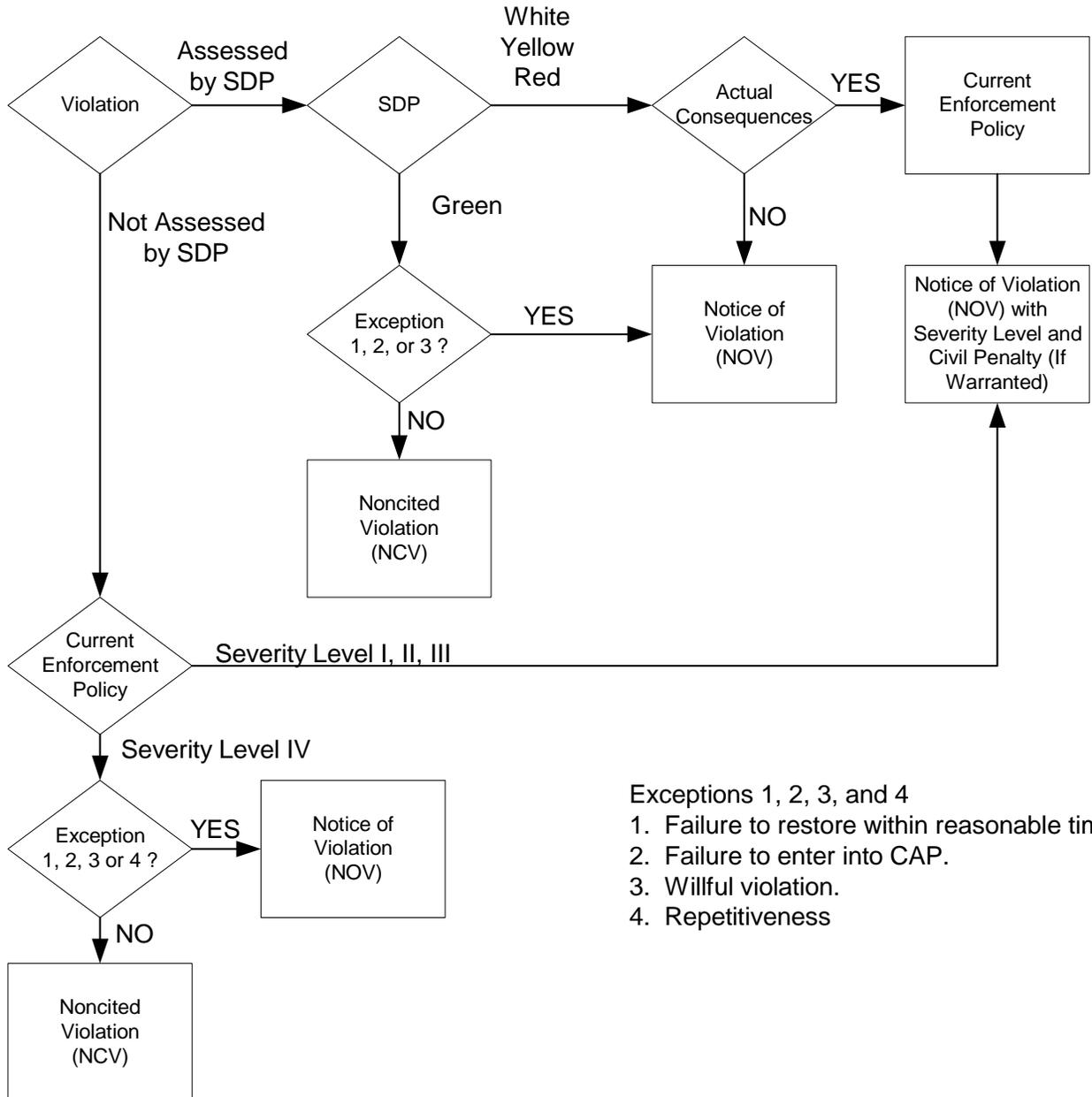
- “DOUBLE COUNTING” OF PERFORMANCE INDICATORS AND INSPECTION FINDINGS
- NON-VIOLATION SIGNIFICANT FINDINGS IN ASSESSMENT AREA



**NOVEMBER 16, 2000, REACTOR OVERSIGHT
PROGRAM REGIONAL PUBLIC MEETING
(PANEL PRESENTATION SLIDES)
Enforcement**



DISPOSITIONING ENFORCEMENT



- Exceptions 1, 2, 3, and 4
1. Failure to restore within reasonable time.
 2. Failure to enter into CAP.
 3. Willful violation.
 4. Repetitiveness

Issues not evaluated by the SDP include issues involving willfulness, discrimination and impact ability for NRC oversight (i.e., reporting, 50.9, and 50.59)

Actual safety consequences include overexposure to the public or plant personnel and failure to make the required notifications that impact the ability of Federal, State, and local agencies to respond to an actual Site Area or General Emergency, transportation event, or a substantial release of radioactive material.



ENFORCEMENT

PERFORMANCE INDICATOR INACCURACIES:

- INTERIM DISCRETION ENFORCEMENT POLICY IN EFFECT FOR INACCURATE PI SUBMITTALS UNTIL JANUARY 31, 2001, IF THEY ARE NON-WILLFUL. VII.B.6 DISCRETION, SPECIAL CIRCUMSTANCES, DOCUMENT IN INSPECTION REPORT.
- POTENTIAL VIOLATIONS OF 10 CFR 50.9 AND ASSESSED IN ACCORDANCE WITH TRADITIONAL ENFORCEMENT POLICY, SUPPLEMENT VII
- INACCURATE OR INCOMPLETE PI DATA SUBMITTED TO THE NRC THAT WOULD NOT HAVE CAUSED A PI CHANGE COLOR-MINOR VIOLATION/NOT DOCUMENTED



ENFORCEMENT

PERFORMANCE INDICATOR INACCURACIES (CONT'D):

- INACCURATE OR INCOMPLETE PI DATA SUBMITTED TO THE NRC THAT WOULD HAVE CAUSED A PI TO CHANGE FROM GREEN TO WHITE - SEVERITY LEVEL IV.
- INACCURATE OR INCOMPLETE PI DATA SUBMITTED TO THE NRC THAT WOULD HAVE CAUSED A PI TO CHANGE FROM GREEN TO EITHER YELLOW OR RED; WHITE TO EITHER YELLOW OR RED; OR YELLOW TO RED - SEVERITY LEVEL III.
- CONSERVATIVE ERRORS IN PI DATA SUBMITTED TO THE NRC (E.G., WHITE TO GREEN) WILL RESULT IN NO ENFORCEMENT.



ENFORCEMENT

CURRENT PROGRAM ISSUES/INITIATIVES:

- OCTOBER 4, 2000, REVISION TO THE ENFORCEMENT POLICY RAISED MAXIMUM, DAILY BASE CIVIL PENALTIES TO \$120,000 FOR REACTOR LICENSEES.
- EVALUATING ENFORCEMENT POLICY AND PROGRAM CHANGES RELATED TO EMPLOYEE DISCRIMINATION - DISCRIMINATION TASK FORCE.
- EVALUATING ENFORCEMENT POLICY CHANGES RELATED TO INDIVIDUAL ENFORCEMENT ACTIONS.



ENFORCEMENT

CURRENT PROGRAM ISSUES/INITIATIVES (CONT'D):

- MANUAL CHAPTER 0609, ATTACHMENT 1 UNDER REVISION TO BETTER CLARIFY SIGNIFICANCE DETERMINATION EVALUATION PROCESS, PROGRAM REQUIREMENTS, AND TIMELINESS.
- COMMISSION RECENTLY CONCLUDED THAT NO CHANGES SHOULD BE MADE TO THE ALLEGATIONS PROGRAM AS A RESULT OF ROP IMPLEMENTATION. HOWEVER, ENFORCEMENT GUIDANCE AND MANUAL CHAPTER 0610* REVISED TO NO LONGER REQUIRE DOCUMENTATION OF MINOR VIOLATIONS ASSOCIATED WITH ALLEGATIONS IN INSPECTION REPORTS.



ENFORCEMENT

PROCESS ISSUES:

- TIMELINESS OF FINAL SIGNIFICANCE DETERMINATION
- TIMELINESS AND IMPACT OF REGULATORY CONFERENCES, LICENSEE SUBMITTALS, AND SIGNIFICANCE DETERMINATION
- APPROPRIATE NRC AND LICENSEE ATTENDANCE AT REGULATORY CONFERENCES
- RESPONSE IN WRITING VERSUS ATTENDANCE AT REGULATORY CONFERENCES
- POST-INSPECTION INFORMATION EXCHANGE (PRA)



ENFORCEMENT

DOCUMENTATION/IMPLEMENTATION ISSUES:

- CLARITY OF FINAL SIGNIFICANCE DETERMINATION/ ENFORCEMENT CORRESPONDENCE
- CLARITY AND COMPLETENESS OF NON-CITED VIOLATIONS TO SUPPORT REGULATORY BASIS OF ISSUES
- CONSISTENCY/PREDICTABILITY OF NON-CITED VIOLATION CRITERIA APPLICATION
- EXPERIENCE WITH NON-CITED VIOLATION DENIAL PROCESS



ENFORCEMENT

POLICY ISSUES:

- EVALUATION OF ISSUES WHICH INVOLVE BOTH THE ROP AND TRADITIONAL ENFORCEMENT PROCESSES (I.E., 50.59 VIOLATION WHICH RESULTS IN A TECHNICAL SPECIFICATION VIOLATION)
- EVALUATING WILLFUL ISSUES WHICH INVOLVE AN UNDERLYING TECHNICAL ISSUE THAT COULD BE ASSESSED UNDER THE SDP
- INDIVIDUAL ENFORCEMENT ACTION VS. LICENSEE ENFORCEMENT IN DELIBERATE CASES



ENFORCEMENT

POLICY ISSUES (CONT'D):

- ASSESSMENT OF FINDINGS/VIOLATIONS WITH “NO COLOR” DESIGNATION



ENFORCEMENT

TIMELINESS REQUIREMENTS:

TIME = 0: INSPECTION EXIT/PRELIMINARY INSPECTION FINDINGS

2-WEEKS
AFTER EXIT: REGIONAL ANALYSIS OF ISSUE AND COORDINATION WITH SENIOR REACTOR ANALYST

3-WEEKS
AFTER EXIT: CONDUCT SIGNIFICANCE DETERMINATION AND ENFORCEMENT REVIEW PANEL (MULTI-OFFICE)



ENFORCEMENT

TIMELINESS REQUIREMENTS (CONT'D):

4-WEEKS

AFTER EXIT: RE-EXIT WITH LICENSEE (IF NECESSARY) AND ISSUE INSPECTION REPORT/REGULATORY CONFERENCE LETTER WITH PRELIMINARY SIGNIFICANCE DETERMINATION

7-WEEKS

AFTER EXIT: LICENSEE SUBMITTAL OF WRITTEN BASIS FOR DISPUTING NRC'S PRELIMINARY SIGNIFICANCE DETERMINATION (OPTIONAL/ENCOURAGED)



ENFORCEMENT

TIMELINESS REQUIREMENTS (CONT'D):

8-WEEKS

**AFTER EXIT: CONDUCT REGULATORY CONFERENCE
(LICENSEE MUST HAVE INSPECTION REPORT AT
LEAST 2 WEEKS PRIOR TO CONFERENCE.)**

90 DAYS

**AFTER EXIT: ISSUE FINAL SIGNIFICANCE DETERMINATION WITH
ASSOCIATED ENFORCEMENT (AGENCY GOAL)**

NRC Inspection Manual Chapter 0305 - Exhibit 5 - ACTION MATRIX

		Licensee Response Column	Regulatory Response Column	Degraded Cornerstone Column	Multiple/ Repetitive Degraded Cornerstone Column	Unacceptable Performance Column
RESULTS		All Assessment Inputs (Performance Indicators (PIs) and Inspection Findings) Green; Cornerstone Objectives Fully Met	One or Two White Inputs (in different cornerstones) in a Strategic Performance Area; Cornerstone Objectives Fully Met	One Degraded Cornerstone (2 White Inputs or 1 Yellow Input) or any 3 White Inputs in a Strategic Performance Area; Cornerstone Objectives Met with Minimal Reduction in Safety Margin	Repetitive Degraded Cornerstone, Multiple Degraded Cornerstones, Multiple Yellow Inputs, or 1 Red Input ¹ ; Cornerstone Objectives Met with Longstanding Issues or Significant Reduction in Safety Margin	Overall Unacceptable Performance; Plants Not Permitted to Operate Within this Band, Unacceptable Margin to Safety
RESPONSE	Regulatory Performance Meeting	None	Branch Chief (BC) or Division Director (DD) Meet with Licensee	DD or Regional Administrator (RA) Meet with Licensee	RA (or EDO) Meet with Senior Licensee Management	Commission meeting with Senior Licensee Management
	Licensee Action	Licensee Corrective Action	Licensee Corrective Action with NRC Oversight	Licensee Self Assessment with NRC Oversight	Licensee Performance Improvement Plan with NRC Oversight	
	NRC Inspection	Risk-Informed Baseline Inspection Program	Baseline and supplemental inspection procedure 95001	Baseline and supplemental inspection procedure 95002	Baseline and supplemental inspection procedure 95003	
	Regulatory Actions	None	Supplemental inspection only	Supplemental inspection only	-10 CFR 2.204 DFI -10 CFR 50.54(f) Letter - CAL/Order	Order to Modify, Suspend, or Revoke Licensed Activities
COMMUNICATION	Assessment Reports	BC or DD review/sign assessment report (w/ inspection plan)	DD review/sign assessment report (w/ inspection plan)	RA review/sign assessment report (w/ inspection plan)	RA review/sign assessment report (w/ inspection plan) Commission Informed	
	Annual Public Meeting	SRI or BC Meet with Licensee	BC or DD Meet with Licensee	RA (or designee) Discuss Performance with Licensee	EDO (or Commission) Discuss Performance with Senior Licensee Management	Commission Meeting with Senior Licensee Management
	INCREASING SAFETY SIGNIFICANCE ----->					

1. It is expected in a few limited situations that an inspection finding of this significance will be identified that is not indicative of overall licensee performance. The staff will consider treating these inspection findings as exceptions for the purpose of determining appropriate actions.

Summary of Meeting Feedback Issues

Performance Indicators (PIs):

In general, the attendees found the PI data reporting system to be appropriate. They acknowledged the ongoing NRC/NEI dialogue on PIs, and expressed support for the recent pilot PI replacement initiative. The following is a synopsis of the issues discussed regarding PIs:

- There were some concerns that changes to PIs were too frequent, while others indicated that they wanted responses to their PI related questions as soon as possible. The panel explained that changes to PIs are done under the methodical process described in draft Manual Chapter 0608. It was recommended that licensees use a formal process to keep better track of the PIs and any changes being made, as well as to maintain appropriate information to support fault exposures. Open communications with the resident was also found to be very helpful in this matter.
- A concern was expressed regarding changes made via “Frequently Asked Questions” (FAQs) which now require counting unavailability time against mitigating system PIs for planned outages of their support systems. One utility representative indicated that this change in guidance is expected to drive PIs on two of its systems to White. Representatives of the NRC program office acknowledged the potential impact of this change and indicated they would work with NEI to reach some resolution.
- A desire was expressed to make the FAQs more generic; thereby facilitating application to more facilities. It was indicated that the PI Task Force was trying to make such an effort where possible, but it was imperative that licensees read the entire FAQ, not just the answer, to make sure it applies to their particular case. It was indicated that NEI 99-02 would be revised to capture the existing FAQs where possible.
- Concern was raised regarding the effort to maintain different PIs under the maintenance rule and the Reactor Oversight Program (ROP), as well as for WANO. Acknowledging the concern, representatives of the NRC program office indicated that WANO was represented on the PI Task Force and that the ultimate goal is to require licensees to report only one set of performance data which could then be used by NRC, INPO, or WANO to construct appropriate PIs.
- A question was asked regarding enforcement of PI errors. Indicating that there is currently enforcement discretion in place, the program office emphasized that as long as there was a good faith effort (i.e., error not willful or due to a program deficiency), enforcement probably would not be warranted. It was reemphasized that quarterly issuance of PIs supports the agency’s public confidence goal; thus, the accuracy and completeness of PI data is essential.
- In general, licensee participants indicated that changing surveillance periods to avoid T/2 hits and increased management attention to plan/schedule equipment out of service time, was part of doing business under the ROP. There was one concern expressed for not being able to take credit for operator action when reporting unavailability under the ROP.

- The possible need for alternate PIs for unplanned power reductions and safety system unavailability was discussed; referencing apparent conflicts with the maintenance rule and the ROP (e.g., thresholds on unavailability, etc.).
- The attendees discussed the relevance of several PI thresholds. The panel explained that the > 5 gpm reactor coolant system leakage Green/White threshold was intentionally set at 50% of the Technical Specification (TS) limit since its purpose was to show system degradation, not a TS required shutdown. The Barrier and Scram PIs were similarly based.
- It was suggested that event classifications performed in the simulator during requalification could mask poor performance during drill/actual event classifications. It was indicated that some licensees have seen just the opposite and that classification calls made in the simulator are as rigorous as those made during drills.
- When questioned, all found the PI data reporting system to be appropriate.

Inspections:

In general, the attendees indicated that the inspection reports under the ROP were well focused on the safety significant issues. The licensee representatives also expressed unanimous support for use of quarterly integrated inspection reports. Specific comments were as follows:

- There was a concern that non-colored issues were being used as another way to document minor or non-regulatory issues. Several participants requested clarification on how the Group 1 questions were being used. The panel explained that entry into the process requires a performance deficiency and the Group 1 questions are used to determine if the issue is minor. Regarding non-colored issues, the panel indicated that such issues should only result from an affirmative answer from a Group 3 "Extenuating Circumstances" question (e.g., willfulness, 10 CFR 50.59, closure of an Licensee Event Report, etc.).
- It was indicated that some of the non-colored issues stemmed from the maintenance rule and that implementation of maintenance rule a (4) would not necessarily correct this. Consequently, several attendees suggested that a separate Significance Determination Process (SDP) was needed for maintenance rule related issues.
- An issue was raised regarding the subjective selection (i.e., "cherry picking") of issues from the licensee's corrective action program. In response, the panel indicated that "cherry picking" wasn't the goal, but that risk significant issues (i.e., White or greater), regardless of who finds them, will be captured under the ROP. The licensee representatives indicated that a risk guided selection of issues from their corrective action program would not hinder self-identification and correction of issues. A point was made that the problem identification and resolution inspections are resource intensive but so far have not identified any significant findings.

- Some of the licensee's indicated that under the new program, they are seeing higher charges (more time) than was anticipated/budgeted for. The NRC indicated that they are still looking at estimated versus actual times to perform the new Inspection Procedures (IPs).
- The question was asked if resident inspector IPs could be scheduled/announced ahead of time, similar to regionally inspected IPs, so licensees can do self-assessments in the respective areas prior to the inspection. In response, it was explained that the resident inspector IPs were more focused on current activities and, as a result, did not lend themselves as well to the advance scheduling that region-based inspections allow. However, it was noted that as the tentative resident IP schedule is developed by the residents, they typically discuss the upcoming inspection schedule during their routine meetings with their licensee contacts. A licensee representative also asked if their self-assessments could be used for credit in satisfying the risk-informed baseline inspection requirements. The panel explained that the baseline program was the minimum inspection oversight by the NRC and self assessments could not now be used to substitute for the NRC's direct inspection under that program.
- The time between the identification of an unresolved item (URI) and its resolution was mentioned to be exceedingly long. It was explained that all Phase II SDPs are performed by the Senior Reactor Analysts (SRAs) and that issues involving such areas as shutdown risk are performed in NRC headquarters. For some issues, Task Interface Agreements (TIAs) are written for NRR's technical assistance, which also adds to the time it takes to resolve an URI. Although the region makes the TIA response available to the licensee at the time of resolution, NRR indicated that they would look into making the TIA available to the licensee during the response process. Licensee representatives expressed a desire to be able to provide their perspectives on issues submitted to NRR via TIAs.
- When questioned, some licensee representatives indicated that inspection reports may not provide enough information from the public perspective. However, most licensee representatives indicated that capturing just the significant issues was appropriate.
- Licensee representatives also indicated that the quarterly frequency of the integrated inspection reports was appropriate as long as "real time" feedback continued.

Significance Determination Process (SDP)

In general, the licensee representatives indicated that the successes associated with use of the SDP outweigh the challenges.

- To respond to several SDP process questions, the NRC program office gave an overview of the SDP, as depicted in Manual Chapter 0609 (e.g., Phase I, Phase II, Phase III, Significance Evaluation Review Panel, Regulatory Conference, etc.). It was indicated that integrating the risk contribution from external events has been a challenge because, in some cases, the risk is unknown and/or not adequately documented.
- When using the Phase II work sheets, several licensee representatives asked why they obtained different answers than the NRC. It was reemphasized that the Phase II work sheets, which were intended to be a simple screening tool to be used by the inspector,

still need to be finalized. Until they are finalized, the SRA will conduct Phase III reviews, when warranted. Once completed, the revised Phase II work sheets, in conjunction with the Reactor Safety risk informed notebooks, should alleviate this problem.

- Regarding other Phase II SDPs, it was indicated that the Commission is considering a staff paper on a revised physical protection SDP that better ties it to the Reactor Safety SDP. This revised SDP should permit resolution of those security-related URIs which are currently held in abeyance. It was also indicated that the Phase II SDP sheets in Fire Protection reflect values for remote shutdown credit that may not be appropriate, as some facilities are better/or worse than others. More attention is needed in this area as well.
- In response to an inquiry about developing a list of FAQs for SDPs, the NRC program office indicated that there were no current plans to do so.
- A concern was raised over the potential for negative public perception stemming from the fact that operator credit is allowed in the SDP, but not in the PIs. It was noted that this could allow an event to result in a PI White while the SDP assesses it as Green. In response, the panel indicated that because PIs measure availability and the SDP reflects significance, it is appropriate to incorporate operator actions in SDP evaluations of the overall significance of an issue.
- Regarding the Group 1, 2, and 3 (Phase I) questions in Manual Chapter 0609, it was indicated that the Group 1 questions were the most subjective; whereas the others were more clear cut. Hence, the need for better explanations on how the questions are answered was expressed. In response, the panel indicated that for those findings that are documented in inspection reports, the intent of Manual Chapter 0610* is to provide sufficient information in the reports to allow the flow path through the process to be repeated.
- When asked, the licensee representatives indicated that the extensive analysis to characterize an issue as Green did not detract from corrective action focus.

Assessment, Enforcement

The attendees made the following observations:

- A concern was raised regarding the accuracy of the ROP Web page (e.g., untimely data entry, etc.). NRC committed to place additional emphasis on the accuracy and timeliness of web page data.
- Under the current policy, willful actions/violations are handled outside the SDP. A question was raised as to whether willful actions that cause a crossing of a PI threshold would be allowed to change the color of the PI. In response, the NRC program office explained that a PI is a measure of system unavailability, whether it is caused by a willful act or not.
- In response to a question regarding any meaning to the downward trend in Non-Cited Violations (i.e., enforcement) under the new program, the panel indicated that the correlation was being monitored, but it was still too early to make any conclusion.

- Recognizing that timeliness of dispositioning greater than Green issues is important due to the fact that it stays in the system for four quarters, a question was raised regarding when the “clock” is initiated for identified issues. In response, the program office indicated that when the NRC makes its final risk determination and issues its report, the issue time line would be linked to the associated inspection report (i.e., the quarter in which the issue was first identified). It was also suggested that there be some consideration for removal of issues when they are corrected, or perhaps grading how long the Green issue remains in the system based on color/significance.

December 8, 2000

Carolina Power and Light Company
ATTN: Mr. C. S. Hinnant, Senior Vice President
Nuclear Generation/Chief Nuclear Officer, CPB12
P. O. Box 1551
Raleigh, NC 27602

SUBJECT: MEETING SUMMARY - REACTOR OVERSIGHT PROGRAM REGIONAL
PUBLIC MEETING

Dear Mr. Hinnant:

This refers to the Reactor Oversight Program meeting conducted on November 16, 2000, at the Sheraton Gateway Hotel in College Park, Georgia. A copy of the meeting agenda, list of attendees, presentation handouts, NRC Inspection Manual Chapter 0305 Action Matrix, and summary of meeting feedback issues are enclosed.

It is our opinion that this meeting was beneficial, in that it provided an opportunity for NRC, licensees, NEI, and the public to discuss the progress, challenges, and successes related to the initial implementation of the Reactor Oversight Program for Region II facilities.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosures will be placed in the NRC Public Document Room.

Should you have any questions concerning this meeting, please contact Mr. Robert Carroll at (404) 562-4511.

Sincerely,

//RA//

Victor M. McCree, Deputy Director
Division of Reactor Projects

Docket Nos. 50-325, 50-324, 50-400, 50-261
License Nos. DPR-71, DPR-62, NPF-63, DPR-23

Enclosures: As stated

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December 8, 2000

Tennessee Valley Authority
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Chief Nuclear Officer and
Executive Vice President
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1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: MEETING SUMMARY - REACTOR OVERSIGHT PROGRAM REGIONAL
PUBLIC MEETING

Dear Mr. Scalice:

This refers to the Reactor Oversight Program meeting conducted on November 16, 2000, at the Sheraton Gateway Hotel in College Park, Georgia. A copy of the meeting agenda, list of attendees, presentation handouts, NRC Inspection Manual Chapter 0305 Action Matrix, and summary of meeting feedback issues are enclosed.

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In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosures will be placed in the NRC Public Document Room.

Should you have any questions concerning this meeting, please contact Mr. Robert Carroll at (404) 562-4511.

Sincerely,

//RA//

Victor M. McCree, Deputy Director
Division of Reactor Projects

Docket Nos. 50-259, 50-260, 50-296, 50-327, 50-328,
50-390, 50-391
License Nos. DPR-33, DPR-52, DPR-68, DPR-77, DPR-79,
NPF-90, Construction Permit No. CPPR-92,

Enclosures: As stated

Enclosures: (See Page 2)

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December 8, 2000

Southern Nuclear Operating Company, Inc.
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P. O. Box 1295
Birmingham, AL 35201

SUBJECT: MEETING SUMMARY - REACTOR OVERSIGHT PROGRAM REGIONAL
PUBLIC MEETING

Dear Mr. Woodard:

This refers to the Reactor Oversight Program meeting conducted on November 16, 2000, at the Sheraton Gateway Hotel in College Park, Georgia. A copy of the meeting agenda, list of attendees, presentation handouts, NRC Inspection Manual Chapter 0305 Action Matrix, and summary of meeting feedback issues are enclosed.

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In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosures will be placed in the NRC Public Document Room.

Should you have any questions concerning this meeting, please contact Mr. Robert Carroll at (404) 562-4511.

Sincerely,

//RA//

Victor M. McCree, Deputy Director
Division of Reactor Projects

Docket Nos. 50-348, 50-364, 50-321, 50-366, 50-424, 50-425
License Nos. NPF-2, NPF-8, DPR-57, NPF-5, NPF-68, NPF-81

Enclosures: As stated

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December 8, 2000

South Carolina Electric & Gas Company
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Vice President, Nuclear Operations
Virgil C. Summer Nuclear Station
P. O. Box 88
Jenkinsville, SC 29065

SUBJECT: MEETING SUMMARY - REACTOR OVERSIGHT PROGRAM REGIONAL
PUBLIC MEETING

Dear Mr. Byrne:

This refers to the Reactor Oversight Program meeting conducted on November 16, 2000, at the Sheraton Gateway Hotel in College Park, Georgia. A copy of the meeting agenda, list of attendees, presentation handouts, NRC Inspection Manual Chapter 0305 Action Matrix, and summary of meeting feedback issues are enclosed.

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Should you have any questions concerning this meeting, please contact Mr. Robert Carroll at (404) 562-4511.

Sincerely,

//RA//

Victor M. McCree, Deputy Director
Division of Reactor Projects

Docket Nos. 50-395
License Nos. NPF-12

Enclosures: As stated

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December 8, 2000

Mr. John P. Cowan, Vice President
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ATTN: Manager Nuclear Licensing (NA1B)
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15760 West Power Line Street
Crystal River, FL 34428-6708

SUBJECT: MEETING SUMMARY - REACTOR OVERSIGHT PROGRAM REGIONAL
PUBLIC MEETING

Dear Mr. Cowan:

This refers to the Reactor Oversight Program meeting conducted on November 16, 2000, at the Sheraton Gateway Hotel in College Park, Georgia. A copy of the meeting agenda, list of attendees, presentation handouts, NRC Inspection Manual Chapter 0305 Action Matrix, and summary of meeting feedback issues are enclosed.

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Should you have any questions concerning this meeting, please contact Mr. Robert Carroll at (404) 562-4511.

Sincerely,

//RA//

Victor M. McCree, Deputy Director
Division of Reactor Projects

Docket No. 50-302
License No. DPR-72

Enclosures: As stated

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December 8, 2000

Florida Power and Light Company
ATTN: Mr. T. F. Plunkett
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Juno Beach, FL 33408-0420

SUBJECT: MEETING SUMMARY - REACTOR OVERSIGHT PROGRAM REGIONAL
PUBLIC MEETING

Dear Mr. Plunkett:

This refers to the Reactor Oversight Program meeting conducted on November 16, 2000, at the Sheraton Gateway Hotel in College Park, Georgia. A copy of the meeting agenda, list of attendees, presentation handouts, NRC Inspection Manual Chapter 0305 Action Matrix, and summary of meeting feedback issues are enclosed.

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Should you have any questions concerning this meeting, please contact Mr. Robert Carroll at (404) 562-4511.

Sincerely,

//RA//

Victor M. McCree, Deputy Director
Division of Reactor Projects

Docket Nos. 50-250, 50-251, 50-335, 50-389
License Nos. DPR-31, DPR-41, DPR-67, NPF-16

Enclosures: As stated

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December 8, 2000

Mr. M. S. Tuckman
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422 South Church Street
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SUBJECT: MEETING SUMMARY - REACTOR OVERSIGHT PROGRAM REGIONAL
PUBLIC MEETING

Dear Mr. Tuckman:

This refers to the Reactor Oversight Program meeting conducted on November 16, 2000, at the Sheraton Gateway Hotel in College Park, Georgia. A copy of the meeting agenda, list of attendees, presentation handouts, NRC Inspection Manual Chapter 0305 Action Matrix, and summary of meeting feedback issues are enclosed.

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Should you have any questions concerning this meeting, please contact Mr. Robert Carroll at (404) 562-4511.

Sincerely,

//RA//

Victor M. McCree, Deputy Director
Division of Reactor Projects

Docket Nos. 50-269, 50-270, 50-287, 50-413, 50-414, 50-369, 50-370
License Nos. DPR-38, DPR-47, DPR-55, NPF-35, NPF-52, NPF-9, NPF-17

Enclosures: As stated

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S. C. Attorney General's Office
Electronic Mail Distribution

Vanessa Quinn
Federal Emergency Management Agency
Electronic Mail Distribution

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DEC

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cc w/encls: (Cont'd)
North Carolina Electric
Membership Corporation
Electronic Mail Distribution

County Manager of York County, SC
Electronic Mail Distribution

Piedmont Municipal Power Agency
Electronic Mail Distribution

Regulatory Compliance Manager (MNS)
Duke Energy Corporation
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County Manager of Mecklenburg County
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