

December 8, 2000

Mr. Robert J. Barrett
Site Executive Officer
New York Power Authority
Indian Point 3 Nuclear Power Plant
Post Office Box 215
Buchanan, NY 10511

SUBJECT: INDIAN POINT 3 -EVALUATED EMERGENCY PREPAREDNESS EXERCISE -
NRC INSPECTION REPORT NO. 05000286/2000-010

Dear Mr. Barrett:

The enclosed report documents an inspection at the Indian Point 3 Nuclear Power Plant. The inspectors evaluated the performance of your emergency response organization during the November 15, 2000, full-participation exercise, and the post-exercise critique as specified in the reactor oversight program. The inspectors discussed the findings of this inspection with Mr. F. Dacimo and other members of your staff on November 16, 2000.

This inspection was an examination of activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection consisted of a selected examination of procedures and representative records, observations of activities, and interviews with personnel.

Based on the results of this inspection, no significant issues were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

Mr. Robert J. Barrett

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Should you have any questions regarding this report, please contact Mr. Richard J. Conte at (610) 337-5183.

Sincerely,

/RA/ Daniel H. Dorman for:

Wayne D. Lanning, Director
Division of Reactor Safety

Docket No. 05000286

License No: DPR-64

Enclosures:

1. Inspection Report No. 05000286/2000-010
2. NRC's Revised Reactor Oversight Process

cc w/encls:

C. D. Rappleyea, Chairman and Chief Executive Officer
E. Zeltmann, President and Chief Operating Officer
J. Knubel, Chief Nuclear Officer and Senior Vice President
F. Dacimo, Plant Manager
H. P. Salmon, Jr., Vice President of Engineering
W. Josiger, Vice President - Special Activities
J. Kelly, Director - Regulatory Affairs and Special Projects
T. Dougherty, Director - Nuclear Engineering
R. Patch, Director - Quality Assurance
G. C. Goldstein, Assistant General Counsel
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Chairman, Standing Committee on Environmental Conservation, NYS Assembly
T. Morra, Executive Chair, Four County Nuclear Safety Committee
Chairman, Committee on Corporations, Authorities, and Commissions
The Honorable Sandra Galef, NYS Assembly
P. D. Eddy, Electric Division, Department of Public Service, State of New York
F. William Valentino, President, New York State Energy Research
and Development Authority
J. Spath, Program Director, New York State Energy Research
and Development Authority
C. Hehl, Incorporated
C. Terry, Niagara Mohawk Power Corporation
R. Toole
R. Schwarz
County Clerk, West Chester County Legislature
A. Spano, Westchester County Executive
R. Bondi, Putnam County Executive
C. Vanderhoef, Rockland County Executive
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Mr. Robert J. Barrett

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U. S. NUCLEAR REGULATORY COMMISSION

REGION I

Docket No: 05000286

License No: DPR- 64

Report No: 05000286/2000-010

Licensee: New York Power Authority
P.O. Box 215
Buchanan, NY 10511

Facility: Indian Point 3 Nuclear Power Plant

Dates: November 14 - 16, 2000

Inspectors: D. Silk, Senior Emergency Preparedness Inspector, DRS (Lead)
N. McNamara, Emergency Preparedness Inspector, DRS
L. James, Resident Inspector, Indian Point 3, DRP
P. Bissett, Senior Operations Engineer
R. Nimitz, Health Physicist, DRS
R. Bores, State Liaison Officer (FEMA RAC Member)

Approved by: Richard J. Conte, Chief
Operational Safety Branch
Division of Reactor Safety

SUMMARY OF FINDINGS

IR 05000286/2000-010, on 11/14-16/2000; New York Power Authority; Indian Point 3 Nuclear Power Plant. Emergency Preparedness exercise.

This inspection was conducted by region based inspectors and the resident inspector. The significance of issues is indicated by their color (green, white, yellow, red) and was determined by the Significance Determination Process (Enclosure 2).

Cornerstone: Emergency Preparedness

- No significant findings were identified.

Report Details

1. REACTOR SAFETY

Cornerstone: Emergency Preparedness (EP)

1EP1 Drill, Exercise, and Actual Events

a. Inspection Scope

The inspectors reviewed:

- The exercise scenario to determine if the exercise would test major elements of the licensee's emergency plan.
- The licensee's biennial full-participation exercise performance by focusing on risk-significant activities in the control room simulator, the technical support center, and the emergency operations facility (EOF). The risk significant areas are emergency classifications, offsite notification, radiological assessment, and protective action recommendations (PARs).
- The licensee's exercise performance in the above mentioned facilities, as well as, the operations support center and the emergency news center.
- The emergency response organization's (ERO) recognition of abnormal plant conditions, classification of emergency conditions, notification of offsite agencies, development of PARs, command and control, communications, utilization of repair and field monitoring teams, and the overall implementation of the emergency plan.
- The post-exercise critique to evaluate the licensee's self-assessment of the exercise.

b. Issues and Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA2 Identification and Resolution of Problems

a. Inspection Scope

The inspectors reviewed licensee findings pertaining to the recent drills, and the last licensee biennial exercise critique to determine if significant performance trends exist and to determine the effectiveness of licensee corrective actions based upon ERO performance during the exercise.

b. Issues and Findings

No findings of significance were identified.

4OA6 Exit Meeting

The inspectors presented the inspection results to Mr. F. Dacimo and other members of your staff at the conclusion of the inspection on November 16, 2000. The licensee had no objections to the NRC findings.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

R. Barrett, Site Executive Officer
F. Dacimo, Plant Manager
A. Grosjean, Senior Emergency Planner
R. Martin, Emergency Plan Engineer
M. Wilson, Emergency Planning Coordinator

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Closed

None

Discussed

None

LIST OF ACRONYMS USED

DEP	Drill and Exercise Performance
EAL	Emergency Action Level
EOF	Emergency Operations Facility
ERO	Emergency Response Organization
GE	General Emergency
PAR	Protective Action Recommendation

ENCLOSURE 2

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revised its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety

- Initiating Events
- Mitigating Systems
- Barrier Integrity
- Emergency Preparedness

Radiation Safety

- Occupational
- Public

Safeguards

- Physical Protection

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: <http://www.nrc.gov/NRR/OVERSIGHT/index.html>.