

MAR 27 1981

Docket Nos. 50-3
and 50-247

Mr. John D. O'Toole
Assistant Vice President
Nuclear Affairs and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Dear Mr. O'Toole:

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D.Collins

The Commission has issued the enclosed Amendment No. 29 to Provisional Operating License No. DPR-5 for Indian Point Station, Unit No. 1, and Amendment No. 60 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. These amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated March 11, 1981.

The amendments make organizational and administrative changes to the Technical Specifications that you developed after reviewing the events leading to the accumulation of water inside the Indian Point Unit No. 2 containment.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:
S. A. Varga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 29 to DPR-5
2. Amendment No. 60 to DPR-26
3. Safety Evaluation
4. Notice of Issuance

cc: w/enclosures
See next page

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Docket



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 27, 1981

Docket Nos. 50-3
and 50-247

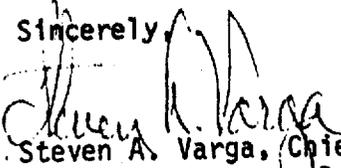
Mr. John D. O'Toole
Assistant Vice President
Nuclear Affairs and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. 29 to Provisional Operating License No. DPR-5 for Indian Point Station, Unit No. 1, and Amendment No. 68 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. These amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated March 11, 1981.

The amendments make organizational and administrative changes to the Technical Specifications that you developed after reviewing the events leading to the accumulation of water inside the Indian Point Unit No. 2 containment.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

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Steven A. Varga, Chief
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cc: w/enclosures
See next page

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Environmental Protection Bureau
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-3

INDIAN POINT STATION, UNIT NO. 1

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 29
License No. DPR-5

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated March 11, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

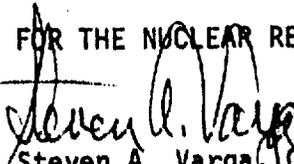
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Provisional Operating License No. DPR-5 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 29, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 27, 1981



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 68
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated March 11, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

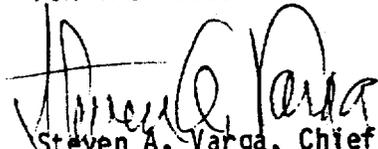
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 68, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 27, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 29
PROVISIONAL OPERATING LICENSE NO. DPR-5
DOCKET NO. 50-3

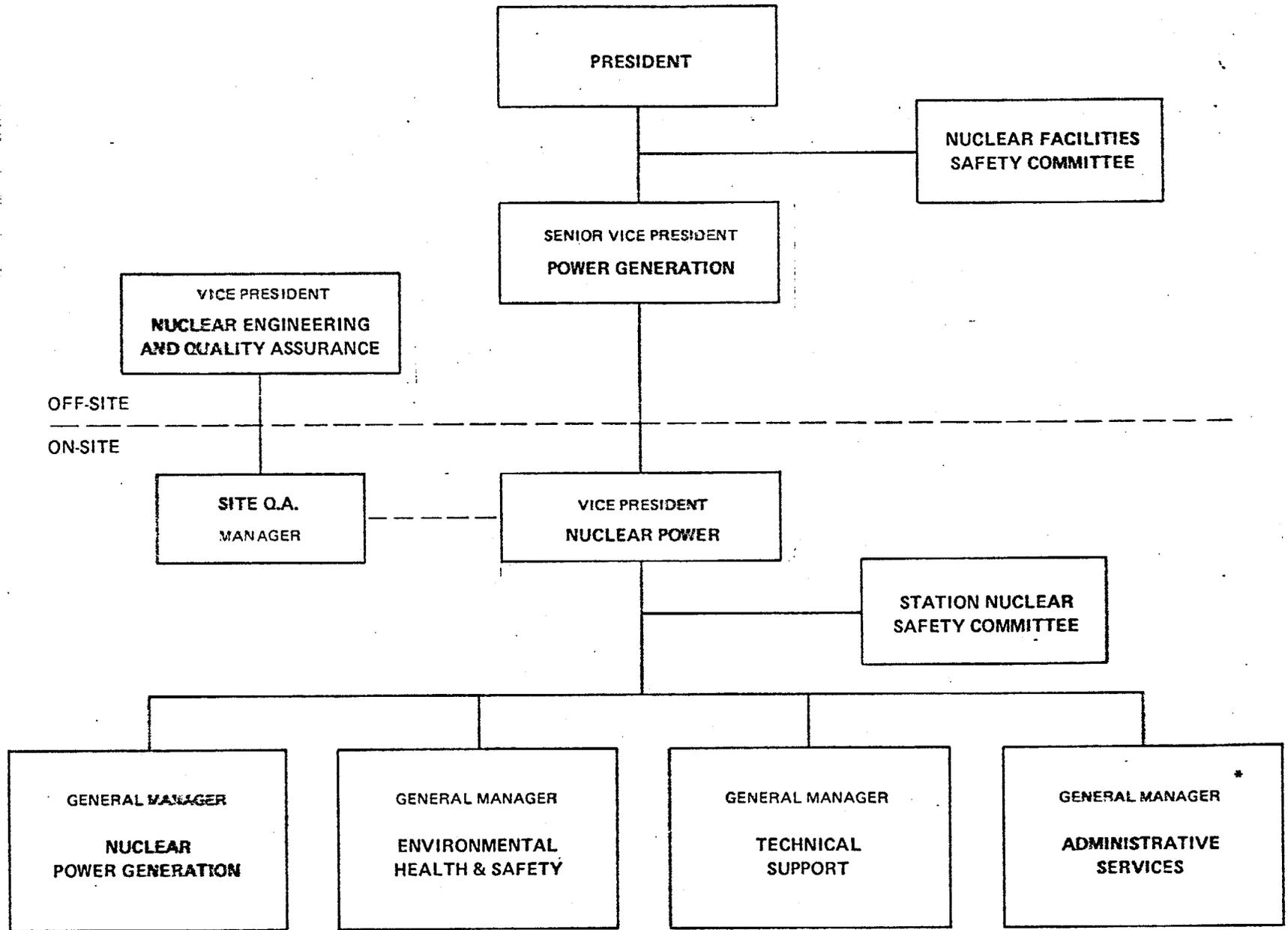
Revise Appendix A as follows:

Remove Pages

3 through 5

Insert Pages

3 through 5



* Responsible for the performance and monitoring of the Fire Protection Program

Figure 3.1 Facility Management and Technical Support Organization

3.0 Administrative and Procedural Safeguards

3.1 Organization

3.1.1 The organization for facility management and technical support shall be as shown in Figure 3.1.

3.1.2 The Facility Organization shall be as shown in Figure 3.2. The Support Facilities Supervisor is responsible for operations at the Unit No. 1 facility.

3.1.3 The Vice President-Nuclear Power shall be responsible for overall facility activities as shown in Figure 3.2 and shall delegate in writing the succession to this responsibility during his absence.

3.1.3.1 The General Manager-Nuclear Power Generation shall be responsible for facility operations as shown in Figure 3.2 and shall delegate in writing the succession to this responsibility during his absence.

3.1.4 The operation of the facility, the operating organization, the procedures for operation, and modifications to the facility shall be subject to review by the Station Nuclear Safety Committee. The committee shall report to the Vice President-Nuclear Power.

3.1.5 The Nuclear Facilities Safety Committee shall function to provide independent review and audit of designated activities in areas of nuclear engineering, chemistry, radiochemistry, metallurgy and non-destructive testing, instrumentation and control, radiological safety, mechanical and electrical engineering, administrative controls and quality assurance practices, and radiological environmental effects.

3.1.6 All fuel handling shall be under the direct supervision of a licensed operator.*

3.2 Operating Instructions and Procedures

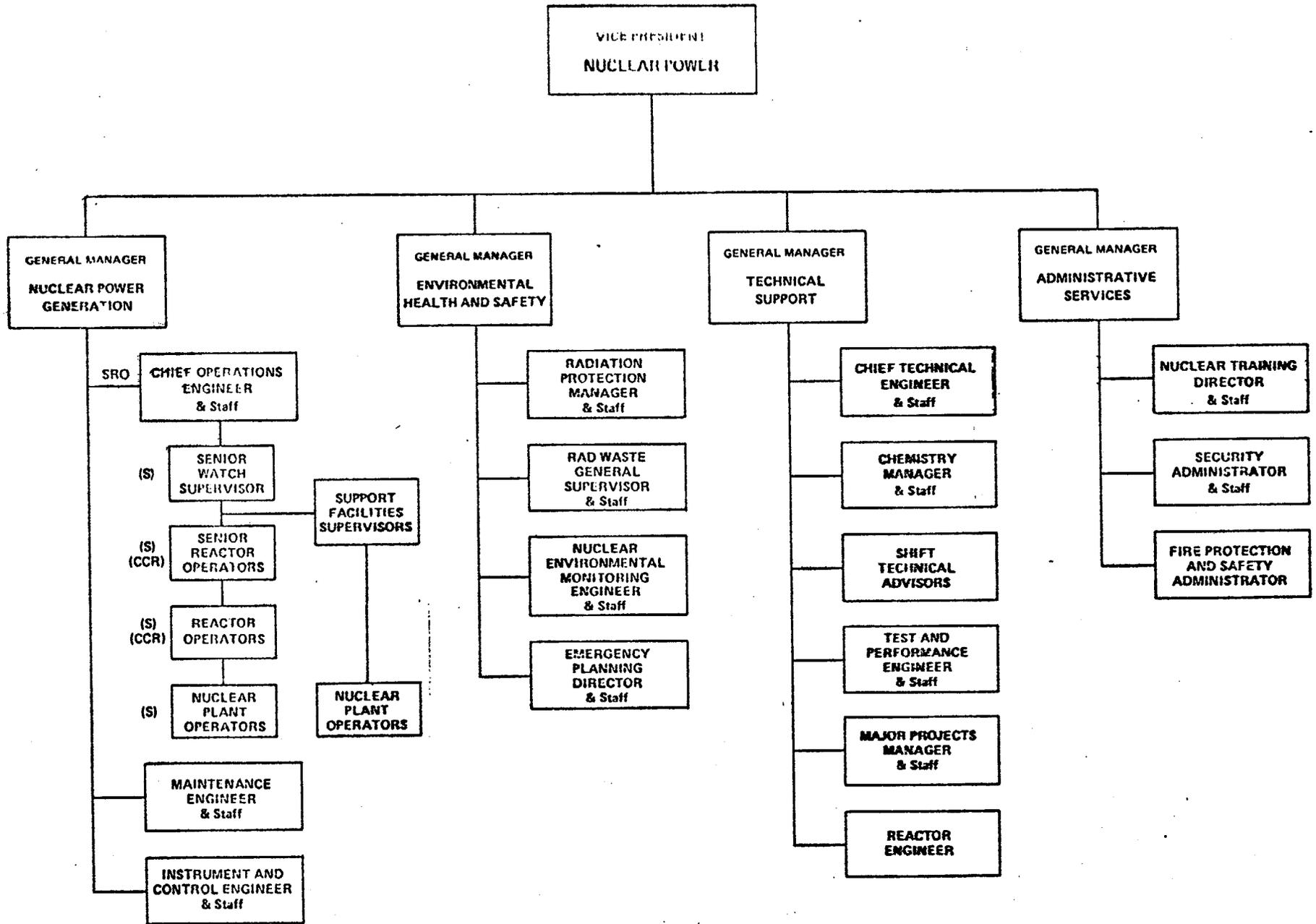
3.2.1 No fuel will be loaded into the reactor core or moved into the reactor containment building without prior review and authorization by the Nuclear Regulatory Commission.

3.2.2 Detailed written instructions setting forth procedures used in connection with the operation and maintenance of the nuclear power plant shall conform to the Technical Specifications.

3.2.3 Operation and maintenance of equipment related to safety when there is no fuel in the reactor shall be in accordance with written instructions.

3.2.6 Radiation control procedures shall be maintained and made available to all station personnel. Station operation shall adhere to these procedures. These procedures show permissible radiation exposure, and shall be consistent with the requirements of 10CFR20. The radiation protection program shall be organized to meet the requirements of 10CFR20.

*Licensed operator for IP-1 or IP-2.



(CCR)- Central Control Room Position
 (S) - Continuous Shift Coverage

Figure 3.2 Facility Organization

ATTACHMENT TO LICENSE AMENDMENT NO. 68

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

6-1 through 6-3
6-5 through 6-9
6-12
6-13

Insert Pages

6-1 through 6-3
6-5 through 6-9
6-12
6-13

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Vice President-Nuclear Power shall be responsible for overall facility activities as shown in Figure 6.2.2 and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The General Manager-Nuclear Power Generations shall be responsible for facility operations as shown in Figure 6.2.2 and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

FACILITY MANAGEMENT AND TECHNICAL SUPPORT

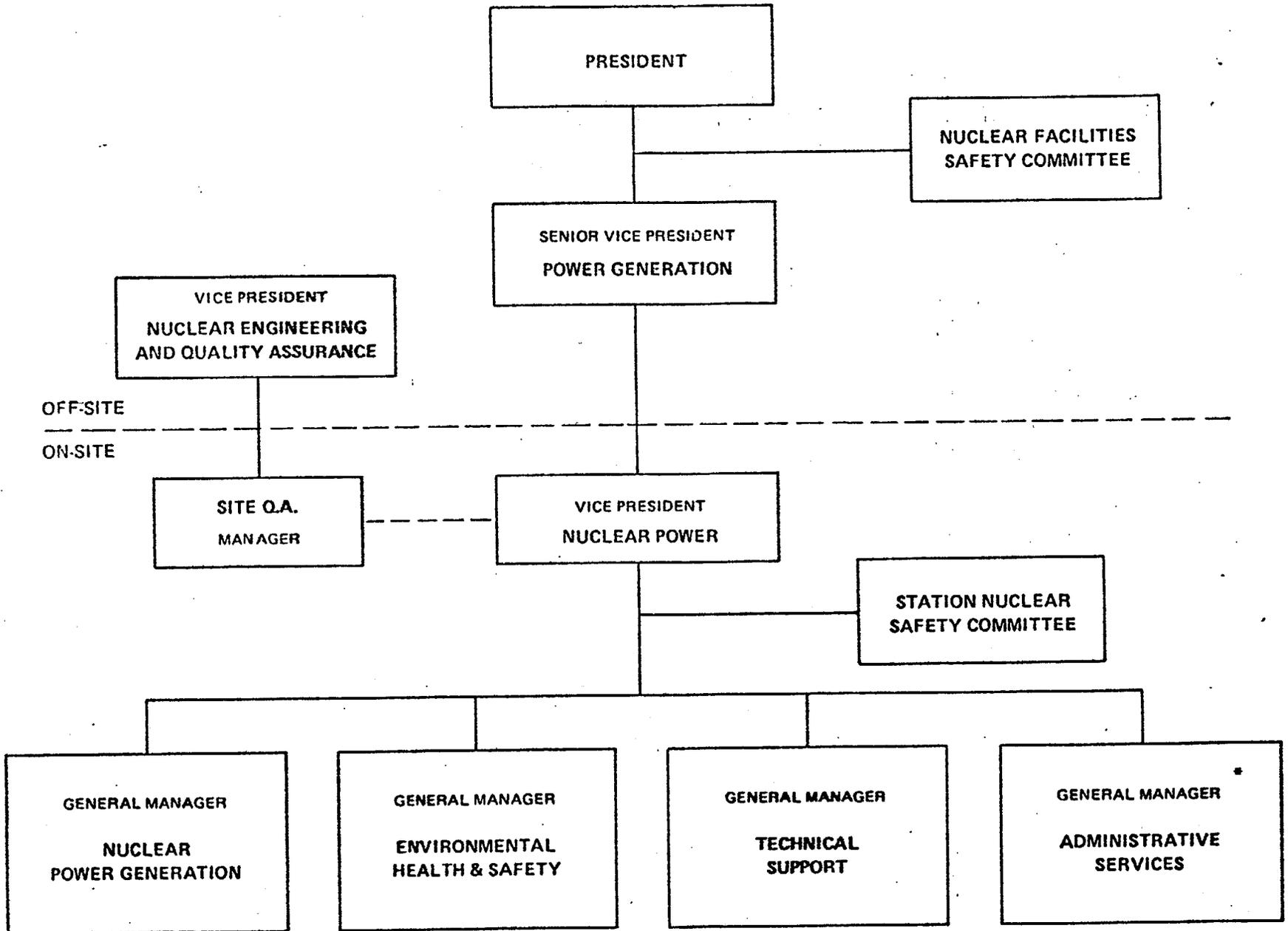
6.2.1 The organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

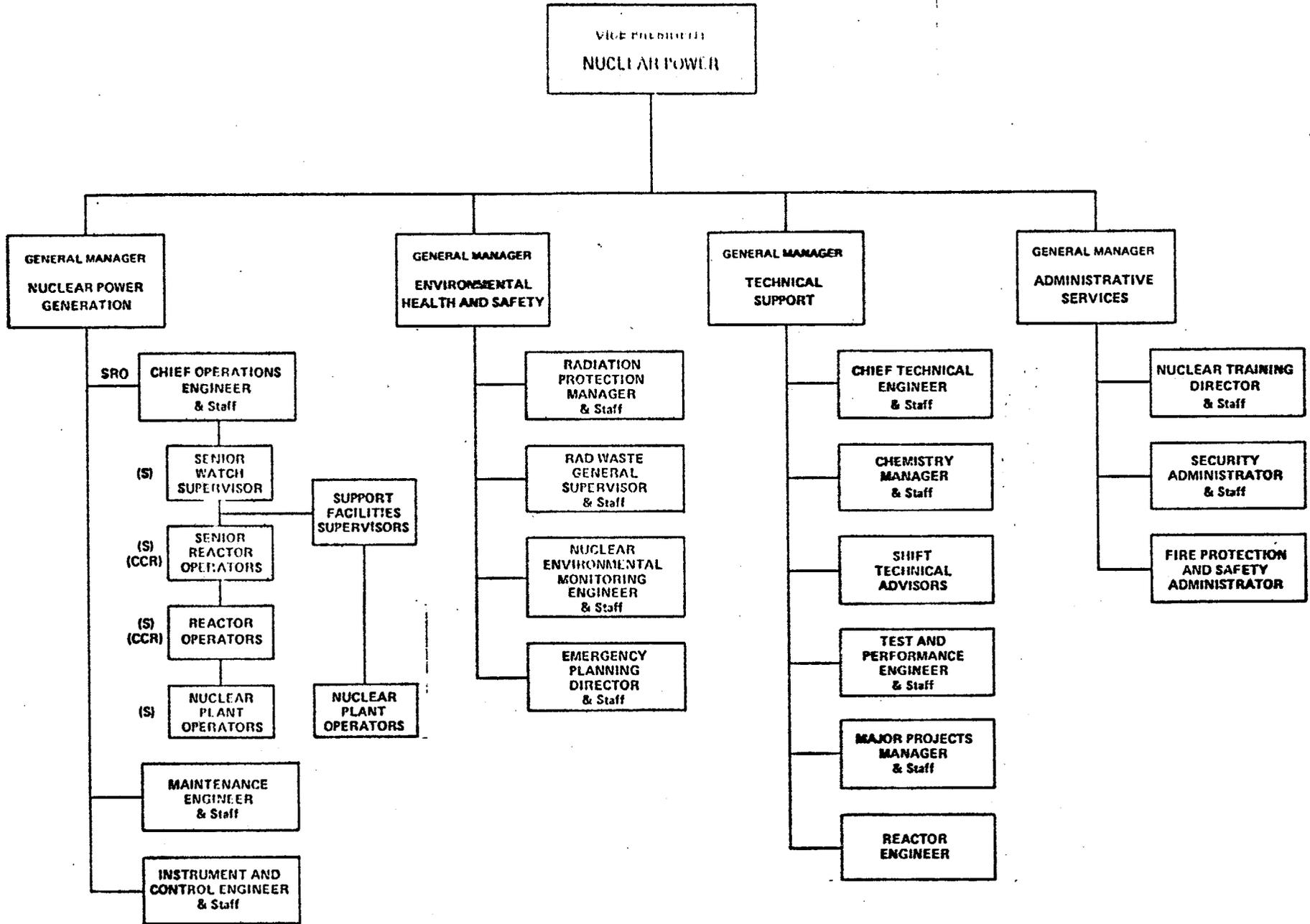
- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor startup, scheduled reactor shutdown, and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling. This individual shall have no other concurrent responsibilities during this operation.
- f. A Fire Brigade of at least five members shall be maintained on site at all times*. This excludes four members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency. During periods of cold shutdown, the Fire Brigade will exclude two members of the minimum shift crew.

*Fire Brigade composition may be one member less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.



*Responsible for the performance and monitoring of the Fire Protection Program

Figure 6.2-1 Facility Management and Technical Support Organization



6.3

(CCR)- Central Control Room Position
(S) - Continuous Shift Coverage

Figure 6.2-2 Facility Organization

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Protection Manager who shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, September 1975.

6.3.2 The General Manager-Nuclear Power Generation shall meet or exceed the minimum qualifications specified for Plant Manager in ANSI N18.1-1971.

6.3.3 The Shift Technical Advisor (STA) shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Director and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Nuclear Training Director and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976 with the exception of the training program schedule.

6.5 REVIEW AND AUDIT

6.5.1 STATION NUCLEAR SAFETY COMMITTEE (SNSC)

FUNCTION

6.5.1.1 The Station Nuclear Safety Committee shall function to advise the Vice President-Nuclear Power on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Nuclear Safety Committee shall, as a minimum, be composed as follows:

| | |
|----------------|---|
| Chairman: | General Manager-Technical Support |
| Vice Chairman: | General Manager-Nuclear Power Generation |
| Member: | General Manager-Administrative Services |
| Member: | General Manager-Environmental Health & Safety |
| Member: | Technical Consultant |
| Member: | Chief Operations Engineer |
| Member: | Maintenance Engineer |
| Member: | Instrument and Control Engineer |
| Member: | Test and Performance Engineer |
| Member: | Major Projects Manager |
| Member: | Reactor Engineer |
| Member: | Security Administrator |
| Member: | Site Q.A. Manager (Q.A. Dept.) |
| Member: | Principal Field Engineer (Eng. Dept.) |

6.5.1.2.1 In addition, other technically competent individuals may be appointed by the SNSC Chairman to serve as SNSC members.

ALTERNATES

6.5.1.3 Alternate members shall be appointed in writing by the SNSC Chairman to serve on a temporary basis, and must have qualifications similar to the member being replaced.

MEETING FREQUENCY

6.5.1.4 The SNSC shall meet at least once per calendar month and as convened by the SNSC Chairman.

QUORUM

6.5.1.5 A quorum of the SNSC shall consist of the Chairman or Vice Chairman and a majority of the committee members including no more than two alternates.

RESPONSIBILITIES

6.5.1.6 The Station Nuclear Safety Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, and 2) any other proposed procedures or changes thereto as determined by the General Manager-Technical Support to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Vice President-Nuclear Power and to the Chairman of the Nuclear Facilities Safety Committee.
- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and the issuance of reports thereon as required by the Chairman of the Nuclear Facilities Safety Committee.
- h. Review of the Plant Security Plan and implementing procedures and submission of recommended changes to the Chairman of the Nuclear Facilities Safety Committee.
- i. Review of the Emergency Plan and implementing procedures and submission of recommended changes to the Chairman of the Nuclear Facilities Safety Committee.

AUTHORITY

6.5.1.7 The Station Nuclear Safety Committee shall:

- a. Recommend to the Vice President-Nuclear Power in writing, approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide immediate written notification to the Chairman, Nuclear Facilities Safety Committee of disagreement between the recommendations of the SNSC and the actions contemplated onsite. However, the course of action determined by the Vice President-Nuclear Power pursuant to 6.1.1 above or the General Manager-Nuclear Power Generation pursuant to 6.1.2 above shall be followed.

RECORDS

6.5.1.8 The Station Nuclear Safety Committee shall maintain written minutes of each meeting and copies shall be provided to, as a minimum, the Vice President-Nuclear Power and the Chairman, Nuclear Facilities Safety Committee.

6.5.2 NUCLEAR FACILITIES SAFETY COMMITTEE (NFSC)

FUNCTION

6.5.2.1 The Nuclear Facilities Safety Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. reactor operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy and non-destructive testing
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. administrative controls and quality assurance practices
- i. radiological environmental effects
- j. other appropriate fields associated with the unique characteristics of the nuclear power plant

COMPOSITION

6.5.2.2 The Committee shall have a permanent voting membership of at least 5 persons of which a majority are independent of the Nuclear Power Organization and shall include technically competent persons from departments of Consolidated Edison having a direct interest in nuclear plant design, construction, operation or in nuclear safety. In addition, persons from departments not having a direct interest in nuclear plant design, construction, operation or nuclear safety may serve as members of the Committee if experienced in the field of nuclear energy. The Chairman and Vice Chairman will be Senior Officials of the Company experienced in the field of nuclear energy.

The Chairman of the Nuclear Facilities Safety Committee, hereafter referred to as the Chairman, shall be appointed by the Chairman of the Board or the President of the Company.

The Vice Chairman shall be appointed by the Chairman of the Board or the President of the Company. In the absence of the Chairman, he will serve as Chairman.

The Secretary shall be appointed by the Chairman of the Committee.

Committee members from departments having a direct interest in nuclear plant design, construction and operation or in nuclear safety shall be designated by the Vice President of the Company who is responsible for the functioning of the department subject to the approval of the Chairman. Committee members from other departments may be appointed by the Chairman with the concurrence of the Vice President of that department.

ALTERNATES

6.5.2.3 Each permanent voting member, subject to the Chairman's approval, may appoint an alternate to serve in his absence. Committee records shall be maintained showing each such current designation.

No more than two alternates shall participate in activities at any one time.

Alternate members shall have voting rights.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NFSC Chairman.

MEETING FREQUENCY

6.5.2.5 The NFSC shall meet at least once per calendar quarter or at more frequent intervals at the call of the Chairman or, in his absence, the Vice Chairman.

QUORUM

6.5.2.6 A majority of the permanent voting committee members, or duly appointed alternates, which shall include the Chairman or the Vice Chairman and of which a minority are from the Nuclear Power Organization shall constitute a quorum for meetings of the committee. In the event both the Chairman and the Vice Chairman are absent, one of the permanent voting members will serve as Acting Chairman.

REVIEW

6.5.2.7 The following subjects shall be reported to and reviewed by the Committee insofar as they relate to matters of nuclear safety:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.

6.6 REPORTABLE OCCURRENCE . ION

6.6.1 The following actions shall be taken in the event of a Reportable Occurrence:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specifications 6.9.1.7.1 and/or 6.9.1.7.2.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the SNSC and submitted to the NFSC Chairman, and the Vice President-Nuclear Power.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit Violation shall be reported to the Commission, the Vice President-Nuclear Power and to the NFSC Chairman immediately.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NFSC Chairman and the Vice President-Nuclear Power within 10 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of USAEC Regulatory Guide 1.33 (issued November 1972) except as provided in 6.8.2 and 6.8.3 below.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and any changes to them shall be reviewed and approved for implementation in accordance with a written administrative control procedure approved by the appropriate General Manager, with the concurrence of the Station Nuclear Safety Committee and the Nuclear Facilities Safety Committee. The administrative control procedure required by this specification shall, as a minimum, require that:

- a. Each proposed procedure/procedure change involving safety related components and/or operation of same receives a pre-implementation review by the SNSC except in case of an emergency.
- b. Each proposed procedure/procedure change which renders or may render the Final Safety Analysis Report or subsequent safety analysis reports inaccurate and those which involve or may involve potential unreviewed safety questions are approved by the SNSC prior to implementation.
- c. The approval of the Nuclear Facilities Safety Committee shall be sought if, following its review, the Station Nuclear Safety Committee finds that the proposed procedure/procedure change either involves an unreviewed safety question or if it is in doubt as to whether or not an unreviewed safety question is involved.

6.8.3 A mechanism shall exist for making temporary changes and they shall only be made by approved management personnel in accordance with the requirements of ANSI 18.7-1972. The change shall be documented, and reviewed by the SNSC within 7 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Region I Office of Inspection and Enforcement unless otherwise noted.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 29 AND 68 TO

OPERATING LICENSE NOS. DPR-5 AND DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT STATION, UNIT NO. 1

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-3 AND 50-247

Introduction

Consolidated Edison Company of New York, Inc. (the licensee), in a letter to the NRC dated March 11, 1981, requested that the Indian Point Unit Nos. 1 and 2 Technical Specifications be modified to reflect certain pending organizational and administrative changes in the management operations and technical support functions for the Indian Point Facility. The licensee stated that it developed the subject changes as part of its commitment to improve its organizational structure and functioning following the accumulation of water inside the Unit 2 containment on October 17, 1980. We have reviewed the proposed changes and have concluded that they are acceptable.

Evaluation

The proposed organizational modifications include the following:

- 1) The current onsite senior manager in overall charge of the facility, the Manager-Nuclear Power Generation Department, would be replaced with a corporate level manager holding the title Vice President-Nuclear Power who would report offsite to the Senior Vice President-Power Generation.
- 2) The quality assurance function that currently reports onsite to the Manager-Nuclear Power Generation Department will report offsite to the Vice President-Nuclear Engineering. This should provide greater independence from the operations organization.
- 3) The other three functions currently reporting to the Manager-Nuclear Power Generation, i.e. technical

engineering, nuclear training and operations (Plant Manager), would be reorganized under four General Managers: Nuclear Power Generation, Environmental Health and Safety, Technical Support, and Administrative Services.

- 4) The functions of the Security Supervisor, Refueling Engineer, Test Performance Engineer, and Chemistry and Radiation Safety Director that currently report to the Plant Manager would be reorganized under the new General Managers for Environmental Health and Safety, Technical Support, and Administrative Services. The new General Manager-Nuclear Power Generation would replace the existing Plant Manager and would manage the operations, maintenance and the instrumentation and control functions. This would reduce the number of functions, as compared to the number managed by the current Plant Manager, that the new General Manager-Nuclear Power Generation would be required to manage and should permit him to direct a quarter portion of his effort to managing operations maintenance and Instrument and Control activities.
- 5) The current chemistry and radiation safety function would be split into three parts; a chemistry function reporting to the General Manager-Technical Support, and a radiation protection and radwaste function reporting to the General Manager-Environmental Health and Safety. This should provide a greater individual visibility to each of these functions.

None of the proposed changes listed above, in our judgement, would diminish the operational safety of the plant. Rather, we believe that they would bring greater management focus on the operation and support activities, and thereby enhance the safe operation of the plant. Accordingly, we conclude that the proposed organizational changes are acceptable.

The composition of the Station Nuclear Safety Committee has been changed to reflect the changes in the organizational structure. The same disciplines would be represented on the proposed committee as are currently represented by the membership of the existing committee. The proposed chairman, however, is the new General Manager-Technical Support - whereas the existing chairman is the Plant Manager. We find that these changes do not diminish the ability of the committee with respect to assuring safe operation of the facility and accordingly conclude they are acceptable.

We have also reviewed all other changes in the text proposed by the March 11, 1981 letter from Con Ed and find that they do not diminish the ability of organization to assure safe operation of the facility and are acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 27, 1981.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-3 AND 50-247CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.NOTICE OF ISSUANCE OF AMENDMENTS TO
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 29 to Provisional Operating License No. DPR-5, and Amendment No. 68 to Facility Operating License No. DPR-26, issued to the Consolidated Edison Company of New York, Inc. (the licensee), which revised Technical Specifications for operation of the Indian Point Station, Unit No. 1 and the Indian Point Nuclear Generating Unit No. 2 (the facilities) located in Buchanan, Westchester County, New York. The amendments are effective as of the date of issuance.

The amendments make organizational and administrative changes to the Technical Specifications that the licensee developed after reviewing events leading to the accumulation of water inside the Indian Point 2 containment.

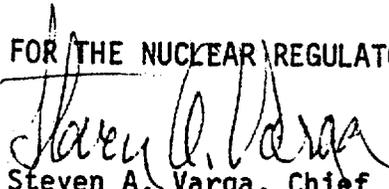
The application for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments did not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated March 11, 1981, (2) Amendment Nos. 29 and 68 to License No. DPR-5 and No. DPR-26, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 27th day of March, 1981

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing