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July 6, 1983

Docket No. 50-247

Mr. John D. O'Toole
Vice President
Nuclear Engineering and Quality Assurance
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Dear Mr. O'Toole:

The Commission has issued the enclosed Amendment No. 84 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated April 28, 1983.

The amendment modifies your Technical Specifications to permit, on a one-time only basis, an extension of 48 hours to the current 24 hour limitation of operation with containment spray pump 21 out of service.

This amendment supports our letter to you dated April 28, 1983, wherein we confirmed our telephone authorization to you of the same date.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:
S. A. Varga

Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:

1. Amendment No. 84 to DPR-26
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 84
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated April 28, 1983; complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

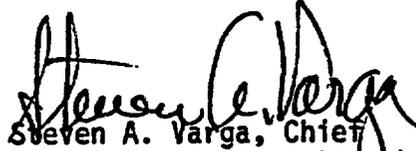
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 84, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective on April 28, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 6, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 84

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Page

Insert Page

3.3-4

3.3-4

requirements of 3.3-1 within the time period specified, the reactor shall be placed in the hot shutdown condition utilizing normal operating procedures. If the requirements of 3.3.3-1 are not satisfied within an additional 48 hours, the reactor shall be placed in the cold shutdown condition utilizing normal operating procedures.

- a. Fan cooler unit 23, 24, or 25 may be non-operable during normal reactor operation for a period not to exceed 24 hours, provided both containment spray pumps are demonstrated to be operable.

OR

Fan cooler unit 21 or 22 may be non-operable during normal reactor operation for a period not to exceed 7 days provided both containment spray pumps are demonstrated daily to be operable.

- b. One containment spray pump may be out of service during normal reactor operation, for a period not to exceed 24 hours,* provided the five fan cooler units are operable and the remaining containment spray pump is demonstrated to be operable.
- c. Any valve required for the functioning of the system during and following accident condition may be inoperable provided it is restored to operable status within 24 hours and all valves in the system that provide the duplicate function are demonstrated to be operable.

C. Isolation Valve Seal Water System (IVSWS)

1. The reactor shall not be brought above cold shutdown unless the following requirements are met:
 - a. The IVSWS shall be operable.
 - b. The IVSW tank shall be maintained at a minimum pressure of 52 psig and contain a minimum of 144 gallons of water.

* One time only exemption for spray pump 21, the 24 hour action statement for operability may be extended for a period of 48 hours ending 1:00 PM April 30, 1983.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 84 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

Introduction

By telecopy request dated April 28, 1983, Consolidated Edison Company of New York, Inc. (the licensee) requested a temporary change to the Indian Point Unit No. 2 Technical Specifications. This amendment would grant on a one-time-only basis modifications to the Technical Specifications to allow an extension of forty-eight (48) hours to the current twenty-four (24) hour limitation with containment spray pump 21 out of service. The requested additional time is to preclude a reactor shutdown and the associated cooldown and cycling of the reactor coolant system and permit sufficient time for repair of the pump. A proposed Technical Specification page revision is attached.

Discussion

Containment cooling at Indian Point Unit No. 2 consists of 5 fan cooler units and two trains of containment spray lines. Unit No. 2 is served by three diesel generators. The diesel generators support these components as follows:

<u>Diesel Generator No.</u>	<u>Components Served</u>
21	#21 spray pump #21 & 22 fan cooler
22	#23 & 24 fan cooler
23	#22 spray pump #25 fan cooler

An evaluation of maintaining the reactor at full power with one spray pump out of service (No. 21) beyond the twenty-four (24) hours permitted by Technical Specification 3.3.B has been performed. This evaluation considers both the peak pressure and the iodine release aspects of the containment analysis.

With containment spray pump 21 out of service and assuming the most limiting single failure of diesel generator 23, four Fan Cooler Unit's (FCU's) would still be available for containment cooling (versus the FSAR analysis assumptions of two spray pumps or five FCU's or one spray train and three FCU's). The licensee's calculations indicate that the containment design pressure of 47 psig will not be exceeded if four FCU's are available. This calculation assumes that the river water temperature is at 85°F. However, the current river water temperature is 46°F. This increases the heat exchanger effectiveness such that the actual heat removal capability of four (4) FCU's with a 46°F river water temperature is at least equal to the heat removal capability of five (5) FCU's at design conditions. Furthermore, while the conservative assumptions above consider no containment spray pumps, recirculation spray capability is unaffected and will be available with FCU's during long-term recirculation phase.

Although the capability to remove certain amounts of iodine activity from the containment is reduced under these "worst-case" assumptions, maintaining the containment integrity will ensure that thyroid doses are within the limits of 10 CFR 100. Since the licensee's analysis demonstrates that containment pressure remains below its design limit, integrity is insured. Furthermore, the Isolation Valve Seal Water System and the Weld Channel and Penetration Pressurization System, which are operable, provide an added level of defense not previously taken credit for in accident analysis and would further limit actual containment leakage. Therefore, the license states that thyroid doses, without injection spray iodine removal capability, remain within the limits of 10 CFR 100. In addition, the FCU's charcoal filtration banks will still be available to remove iodine from containment atmosphere.

The licensee's evaluation of the containment peak pressure analysis and the iodine removal capability is based on the assumed worst case failure of diesel generator 23. However, if this diesel generator is assumed operable, additional containment cooling and iodine removal capability will be present. The licensee's experience with diesel generator testing at Indian Point Unit No. 2 has been excellent. The staff, through its generic program on station blackout and diesel generator reliability, considers the reliability of the Indian Point Unit No. 2 diesel generator to be above average.

Discussion

The Limiting Condition for Operation (LCO) for containment cooling and iodine removal systems is based on having available the capability to remove sufficient amounts of energy and iodine activity from containment following the design basis loss of coolant accident. This LCO ultimately serves to ensure that containment pressure remains below its design value. Maintaining the containment integrity will ensure that thyroid doses to personnel will remain within the limits of 10 CFR 100.

The licensee's basis for this Technical Specification change includes a statement that a reevaluation of the FSAR analysis was conducted using conservative methods and it revealed that the reduced cooling capability is still sufficient to maintain pressure below 47 psig. The details of the reevaluation were discussed by telephone with the licensee. The licensee's methodology and inputs are considered conservative and, therefore, provides a reasonable basis to justify this Technical Specification change.

The staff has also compared the licensee's custom Technical Specifications with NRC's Standard Technical Specifications (STS), recommended for Westinghouse Pressurized Water Reactors (NUREG-0452 Rev. 4), for containment spray pumps. We find the proposed change would be equivalent to the NRC's Standard Technical Specifications. Furthermore, based on the results of the Indian Point Probabilistic Safety Study (IPPSS), the impact of the inoperable spray pump on risk is negligible. Therefore, based in our review of the licensee's submittal we conclude that the proposed one time only Technical Specification change is acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 28, 1983

Principal Contributors:
Douglas Pickett
Roger Pederson
Phil Polk

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-247CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 84 to Facility Operating License No. DPR-26, issued to the Consolidated Edison Company of New York, Inc. (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 2 (the facility) located in Buchanan, Westchester County, New York. The amendment was effective April 28, 1983.

The amendment modifies the Technical Specifications to permit, on a one time only basis, an extension of forty-eight (48) hours to the current twenty-four (24) hour limitation of operation with containment spray pump 21 out of service.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration, and the amendment was authorized prior to effective date of the Commission's Interim Final Rule relating to Notice and State Consultation (48 FR 14873).

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated April 28, 1983, (2) the Commission's letter to the licensee dated April 29, 1983, (3) Amendment No. 84 to License No. DPR-26, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 6th day of July 1983.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing