

January 4, 1977

Dockets Nos.: 50-247 ✓  
and 50-286

Consolidated Edison Company  
of New York, Inc.  
ATTN: Mr. William J. Cahill, Jr.  
Vice President  
4 Irving Place  
New York, New York 10003

Gentlemen:

The Commission has issued the enclosed Amendment No. 26 to Facility Operating License No. DPR-26 and Amendment No. 5 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Units Nos. 2 and 3, respectively. These amendments consist of changes to the Technical Specifications for each license in response to your applications transmitted by letters dated August 27, 1976 and November 9, 1976. As discussed with your staff, modifications have been made to your proposed changes to meet regulatory requirements.

These amendments revise the Technical Specifications to change requirements for administrative controls.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

Original Signed by

Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors

Enclosures:

1. Amendment No. 26 to License No. DPR-26
2. Amendment No. 5 to License No. DPR-64
3. Safety Evaluation
4. Federal Register Notice

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cc w/enclosures: See next page

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DATE >	12/27/76	12/27/76	12/3/77	12/14/77	

Consolidated Edison Company  
of New York, Inc.

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

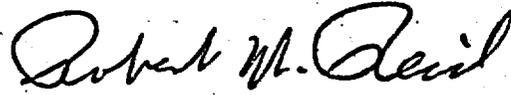
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 26  
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
- A. The applications for amendment by Consolidated Edison Company of New York, Inc. (the licensee) sworn to August 26, 1976 and November 8, 1976, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: January 4, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 26

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Pages

6-1 thru 6-9

6-11

6-15 & 6-16

6-19 & 6-20

Insert Pages

6-1 thru 6-9

6-11

6-15 & 6-16

6-19 & 6-20

Changes on the revised pages are shown by marginal lines.

Pages 6-1, 6-4, 6-15, and 6-20 are unchanged and are included for convenience only.

## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

### 6.2 ORGANIZATION

#### FACILITY MANAGEMENT AND TECHNICAL SUPPORT

6.2.1 The organization for facility management and technical support shall be as shown on Figure 6.2-1.

#### FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling. This individual shall have no other concurrent responsibilities during this operation.

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

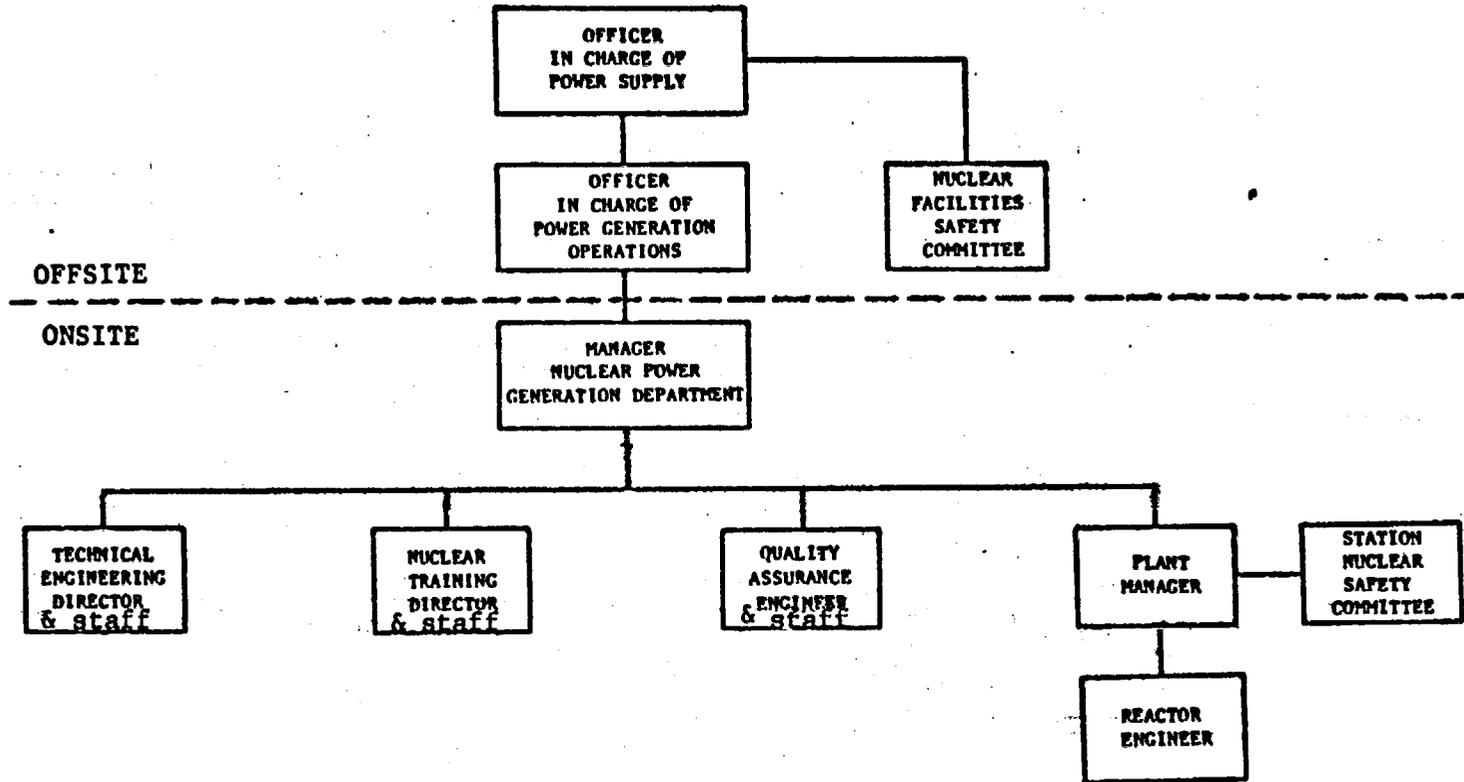


Figure 6.2-1 Facility Management and Technical Support Organization

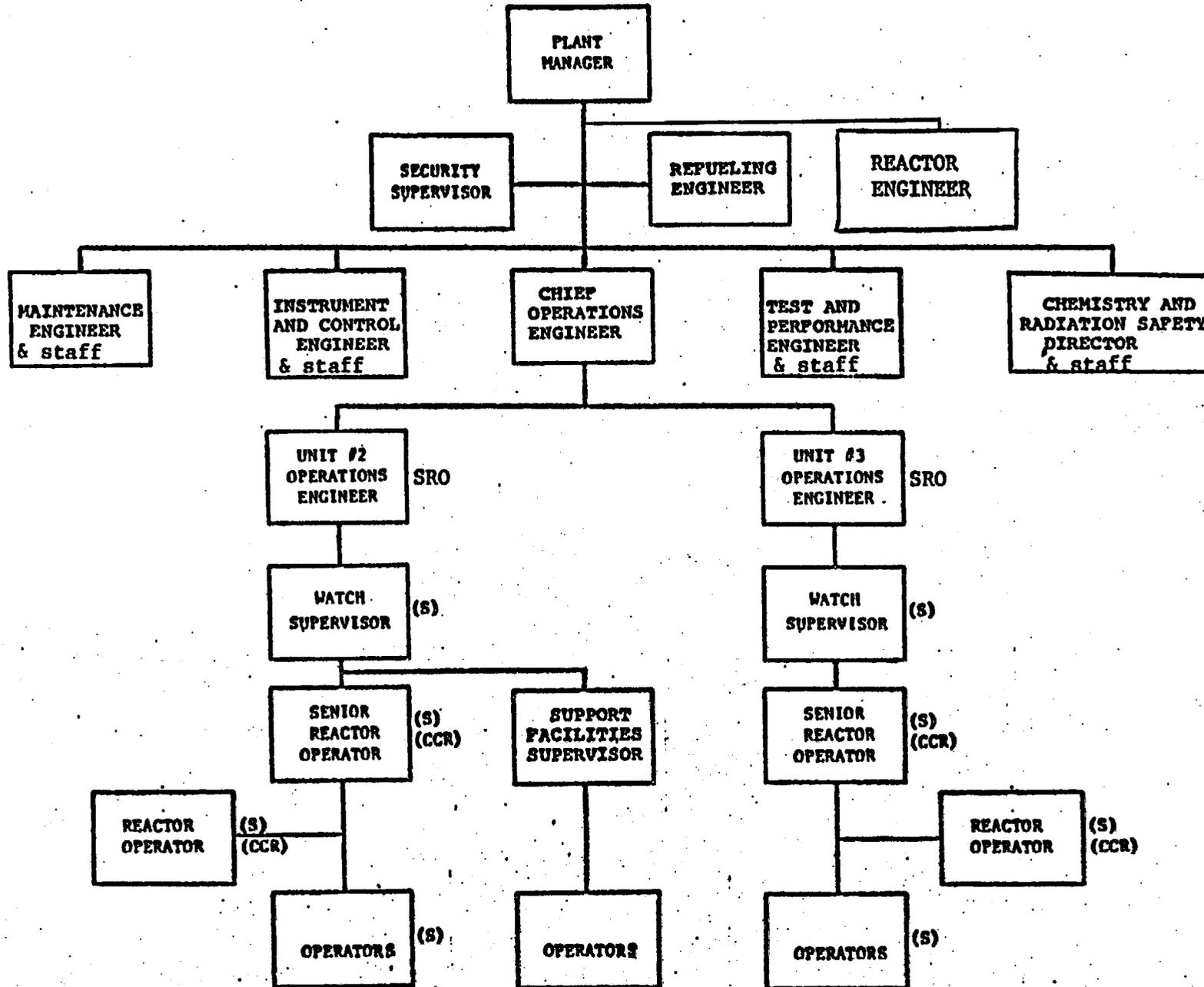


Figure 6.2-2 Facility Organization

(CCR) - Central Control Room Position  
 (S) - Continuous Shift Coverage

Table 6.2-1

Minimum Shift Crew Composition

License Category	During Operations Involving Core Alterations	During Cold Shutdown or Refueling Periods	At All Other Times
Senior Operator License	2*	1	1
Operator License	1	1	2
Non-Licensed	(As Required)	1	2

10

\*Includes individual with SRO license supervising fuel movement as per Section 6.2.2(e).

## 6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Director and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

## 6.5 REVIEW AND AUDIT

### 6.5.1 STATION NUCLEAR SAFETY COMMITTEE (SNSC)

#### FUNCTION

6.5.1.1 The Station Nuclear Safety Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

6.5.1.2 The Station Nuclear Safety Committee shall be composed as follows:

Chairman:	Plant Manager
Member:	Technical Engineering Director
Member:	Quality Assurance Engineer
Member:	Chief Operations Engineer
Member:	Security Supervisor
Member:	Test and Performance Engineer
Member:	Instrument and Control Engineer
Member:	Maintenance Engineer
Member:	Chemistry and Radiation Safety Director
Member:	Reactor Engineer
Member:	Manager NPD

#### ALTERNATES

6.5.1.3 Alternate members shall be appointed in writing by the SNSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in SNSC activities at any one time

#### MEETING FREQUENCY

6.5.1.4 The SNSC shall meet at least once per calendar month and as convened by the SNSC Chairman.

## QUORUM

6.5.1.5 A quorum of the SNSC shall consist of the Chairman or Vice Chairman and five members including alternates.

## RESPONSIBILITIES

6.5.1.6 The Station Nuclear Safety Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, and 2) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Manager, Nuclear Power Generation Department and to the Chairman of the Nuclear Facilities Safety Committee.
- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and the issuance of reports thereon as requested by the Chairman of the Nuclear Facilities Safety Committee.
- h. Review of the Plant Security Plan and implementing procedures and submission of recommended changes to the Chairman of the Nuclear Facilities Safety Committee.
- i. Review of the Emergency Plan and implementing procedures and submission of recommended changes to the Chairman of the Nuclear Facilities Safety Committee.

## AUTHORITY

6.5.1.7 The Station Nuclear Safety Committee shall:

- a. Recommend to the Plant Manager, in writing, approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.

AUTHORITY (Continued)

- c. Provide immediate written notification to the Chairman, Nuclear Facilities Safety Committee and the Manager, Nuclear Power Generation Department of disagreement between the recommendations of the SNSC and the actions contemplated by the Plant Manager. However, the course of action determined by the Plant Manager pursuant to 6.1.1 above shall be followed.

RECORDS

6.5.1.8 The Station Nuclear Safety Committee shall maintain written minutes of each meeting and copies shall be provided to, as a minimum, the Manager, Nuclear Power Generation Department and the Chairman, Nuclear Facilities Safety Committee.

6.5.2 NUCLEAR FACILITIES SAFETY COMMITTEE (NFSC)

FUNCTION

6.5.2.1 The Nuclear Facilities Safety Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. reactor operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices
- i. environmental effects
- j. other appropriate fields associated with the unique characteristics of the nuclear power plant

## COMPOSITION

6.5.2.2 The Committee shall have a permanent voting membership of at least 5 persons of which a majority are independent of the Nuclear Power Generation Department and shall include technically competent persons from departments of Consolidated Edison having a direct interest in nuclear plant design, construction, operation or in nuclear safety. In addition, persons from departments not having a direct interest in nuclear plant design, construction, operation or nuclear safety may serve as members of the Committee if experienced in the field of nuclear energy. The Chairman and Vice Chairman will be Senior Officials of the Company experienced in the field of nuclear energy.

The Chairman of the Nuclear Facilities Safety Committee, hereafter referred to as the Chairman, shall be appointed by the Chairman of the Board or the President of the Company.

The Vice Chairman shall be appointed by the Chairman of the Board or the President of the Company. In the absence of the Chairman, he will serve as Chairman.

The Secretary shall be appointed by the Chairman of the Committee.

Committee members from departments having a direct interest in nuclear plant design, construction and operation or in nuclear safety shall be designated in writing by the Vice President of the Company who is responsible for the functioning of the department subject to the approval of the Chairman. Committee members from other departments may be appointed by the Chairman with the concurrence of the Vice President of that department.

## ALTERNATES

6.5.2.3 Each permanent voting member may appoint an alternate to serve in his absence. Committee records shall be maintained showing each such current designation.

No more than two alternates shall participate in activities at any one time.

Alternate members shall have voting rights.

## CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NFSC Chairman.

## MEETING FREQUENCY

6.5.2.5 The NFSC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

## QUORUM

6.5.2.6 A majority of the permanent voting committee members, or duly appointed alternates, which shall include the Chairman or the Vice Chairman and of which a minority are from the Nuclear Power Generation Department shall constitute a quorum for meetings of the committee. In the event both the Chairman and the Vice Chairman are absent, one of the permanent voting members will serve as Acting Chairman.

## REVIEW

6.5.2.7 The following subjects shall be reported to and reviewed by the Committee insofar as they relate to matters of nuclear safety:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.

## RECORDS

6.5.2.10 Records of NFSC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NFSC meeting shall be prepared, approved and forwarded to the Senior Company Officer in charge of Power Supply within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 e, f, g and h above, shall be prepared, approved and forwarded to the Senior Company Officer in charge of Power Supply within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Company Officer in charge of Power Supply and to the management positions responsible for the areas audited within 30 days after completion of the audit.

## 6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken in the event of a Reportable Occurrence:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the SNSC and submitted to the NFSC Chairman, and the Manager, Nuclear Power Generation Department.

## 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Commission, the Manager, Nuclear Power Generation Department and to the NFSC Chairman immediately.

RECORD RETENTION (Continued)

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Record and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the SNSC and the NFSC.

4  
6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

## 6.12 RESPIRATORY PROTECTION PROGRAM

### ALLOWANCE

6.12.1 Pursuant to 10 CFR 20.103(c)(1) and (3), allowance may be made for the use of respiratory protective equipment in conjunction with activities authorized by the operating license for this facility in determining whether individuals in restricted areas are exposed to concentrations in excess of the limits specified in Appendix B, Table I, Column 1, of 10 CFR 20, subject to the following conditions and limitations:

- a. The limits provided in Section 20.103(a) and (b) shall not be exceeded.
- b. If the radioactive material is of such form that intake through the skin or other additional route is likely, individual exposures to radioactive material shall be controlled so that the radioactive content of any critical organ from all routes of intake averaged over 7 consecutive days does not exceed that which would result from inhaling such radioactive material for 40 hours at the pertinent concentration values provided in Appendix B, Table I, Column 1, of 10 CFR 20.
- c. For radioactive materials designated "Sub" in the "Isotope" column of Appendix B, Table I, Column 1 of 10 CFR 20, the concentration value specified shall be based upon exposure to the material as an external radiation source. Individual exposures to these materials shall be accounted for as part of the limitation on individual dose in 20.101. These materials shall be subject to applicable process and other engineering controls.

### PROTECTION PROGRAM

6.12.2 In all operations in which adequate limitation of the inhalation of radioactive material by the use of process or other engineering controls is impracticable, the licensee may permit an individual in a restricted area to use respiratory protective equipment to limit the inhalation of airborne radioactive material, provided:

- a. The limits specified in 6.12.1 above, are not exceeded.
- b. Respiratory protective equipment is selected and used so that the peak concentrations of airborne radioactive material inhaled by an individual wearing the equipment do not exceed the pertinent concentration values specified in Appendix B, Table I, Column 1, of 10 CFR 20. For the purposes of this subparagraph, the concentration of radioactive material that is inhaled when respirators are worn may be determined by dividing the ambient airborne concentration by the protection factor

Amendment No. 26  
6-19

TABLE 6.12-1  
PROTECTION FACTORS FOR RESPIRATORS

Description	MODES <sup>1/</sup>	<u>PROTECTION FACTORS<sup>2/</sup></u> PARTICULATES AND VAPORS AND GASES EXCEPT TRITIUM OXIDE <sup>3/</sup>	<u>GUIDES TO SELECTION OF EQUIPMENT</u> BUREAU OF MINES APPROVAL SCHEDULES* FOR EQUIPMENT CAPABLE OF PROVIDING AT LEAST EQUIVALENT PROTECTION FACTORS *or schedule superseding for equip- ment of type listed
<b>I. <u>AIR-PURIFYING RESPIRATORS</u></b>			
Facepiece, half-mask <sup>4/ 7/</sup>	NP	5	30 CFR Part 11 Subpart K
Facepiece, full <sup>7/</sup>	NP	100	30 CFR Part 11 Subpart K
<b>II. <u>ATMOSPHERE-SUPPLYING RESPIRATOR</u></b>			
<b>1. <u>Airline respirator</u></b>			
Facepiece, half-mask	CF	100	30 CFR Part 11 Subpart J
Facepiece, full	CF	1,000	30 CFR Part 11 Subpart J
Facepiece, full <sup>7/</sup>	D	100	30 CFR Part 11 Subpart J
Facepiece, full	PD	1,000	30 CFR Part 11 Subpart J
Hood	CF	5/	30 CFR Part 11 Subpart J
Suit	CF	5/	30 CFR Part 11 Subpart J 6/
<b>2. <u>Self-contained breathing apparatus (SCBA)</u></b>			
Facepiece, full <sup>7/</sup>	D	100	30 CFR Part 11 Subpart H
Facepiece, full	PD	1,000	30 CFR Part 11 Subpart H
Facepiece, full	R	100	30 CFR Part 11 Subpart H
<b>III. <u>COMBINATION RESPIRATOR :</u></b>			
Any combination of air-purifying and atmosphere supplying respirator		Protection factor for type and mode of operation as listed above.	30 CFR Part 11 § 11.63(b)

1/, 2/, 3/, 4/, 5/, 6/, 7/, (These notes are on the following pages)

TABLE 6.12-1 (Continued)

1/ See the following symbols:

- CF: continuous flow
- D: demand
- NP: negative pressure (i.e., negative phase during inhalation)
- PD: pressure demand (i.e., always positive pressure)
- R: recirculating (closed circuit)

2/ a. For purposes of this specification the protection factor is a measure of the degree of protection afforded by a respirator, defined as the ratio of the concentration of airborne radioactive material outside the respiratory protective equipment to that inside the equipment (usually inside the facepiece) under conditions of use. It is applied to the ambient airborne concentration to estimate the concentration inhaled by the wearer according to the following formula:

$$\text{Concentration Inhaled} = \frac{\text{Ambient Airborne Concentration}}{\text{Protection Factor}}$$

b. The protection factors apply:

- (i) only for trained individuals wearing properly fitted respirators used and maintained under supervision in a well-planned respiratory protective program.
- (ii) for air-purifying respirators only when high efficiency (above 99.9% removal efficiency by U.S. Bureau of Mines type dioctyl phthalate (DOP) test) particulate filters and/or sorbents appropriate to the hazard are used in atmospheres not deficient in oxygen.
- (iii) for atmosphere-supplying respirators only when supplied with adequate respirable air.

3/ Excluding radioactive contaminants that present an absorption or submersion hazard. For tritium oxide approximately half of the intake occurs by absorption through the skin so that an overall protection factor of not more than approximately 2 is appropriate when atmosphere-supplying respirators are used to protect against tritium oxide. Air-purifying respirators are not recommended for use against tritium oxide.

See also footnote 5/, below, concerning supplied-air suits and hoods.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

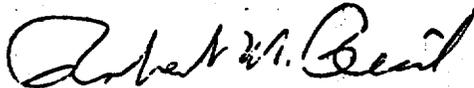
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 5  
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment by Consolidated Edison Company of New York, Inc. and the Power Authority of the State of New York (the licensees) sworn to August 26, 1976 and November 8, 1976, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
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FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: January 4, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 5

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

6-1 thru 6-10

6-13 & 6-14

Insert Pages

6-1 thru 6-10

6-13 & 6-14

Changes on the revised pages are shown by marginal lines.

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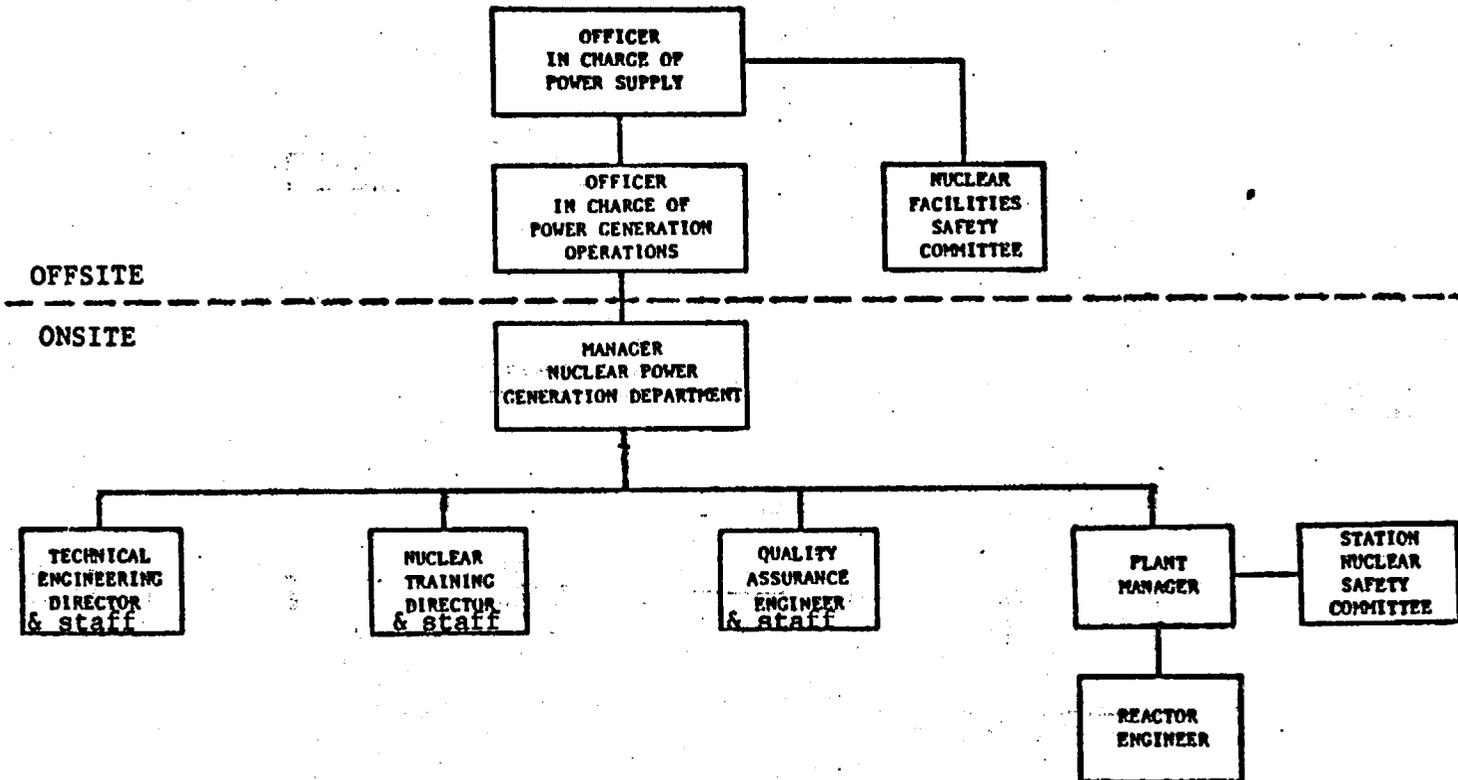


Figure 6.2-1 Facility Management and Technical Support Organization

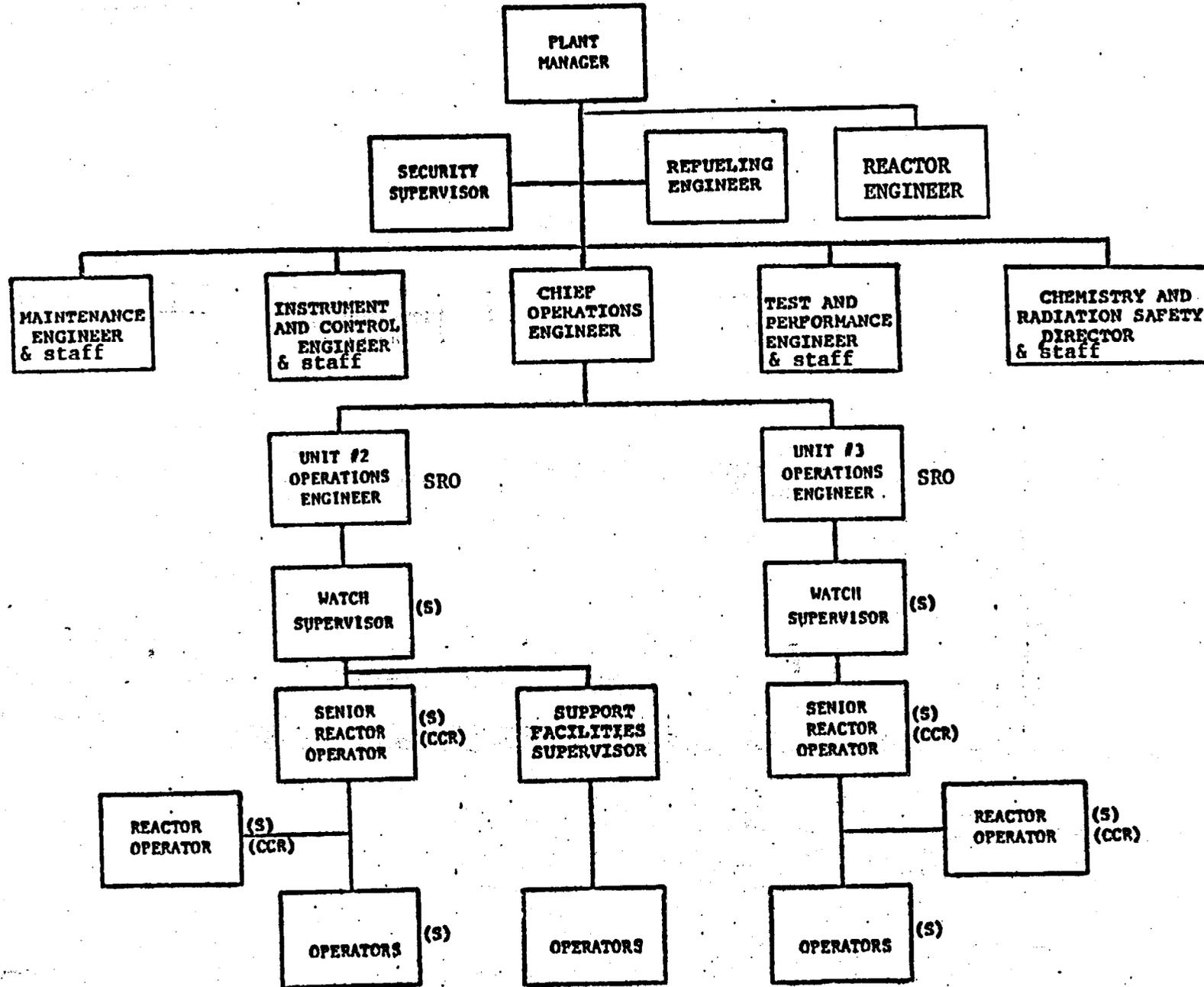


Figure 6.2-2 Facility Organization

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6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Director and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55.

6.5 REVIEW AND AUDIT

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Member:	Instrument and Control Engineer
Member:	Maintenance Engineer
Member:	Chemistry and Radiation Safety Director
Member:	Reactor Engineer
Member:	Manager NPG

Alternates

6.5.1.3 Alternate members shall be appointed in writing by the SNSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in SNSC activities at any one time.

### Meeting Frequency

6.5.1.4 The SNSC shall meet at least once per calendar month and as convened by the SNSC Chairman.

### Quorum

6.5.1.5 A quorum of the SNSC shall consist of the Chairman or Vice Chairman and five members including alternates.

### Responsibilities

6.5.1.6 The Station Nuclear Safety Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, and 2) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Manager, Nuclear Power Generation Department and to the Chairman of the Nuclear Facilities Safety Committee.
- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and the issuance of reports thereon as requested by the Chairman of the Nuclear Facilities Safety Committee.

- h. Review of the Plant Security Plan and implementing procedures and submission of recommended changes to the Chairman of the Nuclear Facilities Safety Committee.
- i. Review of the Emergency Plan and implementing procedures and submission of recommended changes to the Chairman of the Nuclear Facilities Safety Committee.

#### Authority

6.5.1.7 The Station Nuclear Safety Committee shall:

- a. Recommend to the Plant Manager, in writing, approval or disapproval of items considered under 6.5.1.6(a) through (d), above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above, constitutes an unreviewed safety question.
- c. Provide immediate written notification to the Chairman, Nuclear Facilities Safety Committee and the Manager, Nuclear Power Generation Department of disagreement between the recommendations of the SNSC and the actions contemplated by the Plant Manager. However, the course of action determined by the Plant Manager pursuant to 6.1.1 above shall be followed.

#### Records

6.5.1.8 The Station Nuclear Safety Committee shall maintain written minutes of each meeting and copies shall be provided to, as a minimum, the Manager, Nuclear Power Generation Department and the Chairman, Nuclear Facilities Safety Committee.

6.5.2

Nuclear Facilities Safety Committee (NFSC)

Function

6.5.2.1 The Nuclear Facilities Safety Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. reactor operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices
- i. environmental effects
- j. other appropriate fields associated with the unique characteristics of the nuclear power plant

Composition

6.5.2.2 The Committee shall have a permanent voting membership of at least 5 persons of which a majority are independent of the Nuclear Power Generation Department and shall include technically competent persons from departments of Consolidated Edison having a direct interest in nuclear plant design, construction, operation or in nuclear safety. In addition, persons from departments not having a direct interest in nuclear plant design, construction, operation or nuclear safety may serve as members of the Committee if experienced in the field of nuclear energy. The Chairman and Vice Chairman will be Senior Officials of the Company experienced in the field of nuclear energy.

The Chairman of the Nuclear Facilities Safety Committee, hereafter referred to as the Chairman, shall be appointed by the Chairman of the Board or the President of the Company.

The Vice Chairman shall be appointed by the Chairman of the Board or the President of the Company. In the absence of the Chairman, he will serve as Chairman.

The Secretary shall be appointed by the Chairman of the Committee.

Committee members from departments having a direct interest in nuclear plant design, construction and operation or in nuclear safety shall be designated in writing by the Vice President of the Company who is responsible for the functioning of the department subject to the approval of the Chairman. Committee members from other departments may be appointed by the Chairman with the concurrence of the Vice President of that department.

#### Alternates

6.5.2.3 Each permanent voting member may appoint an alternate to serve in his absence. Committee records shall be maintained showing each such current designation.

No more than two alternates shall participate in activities at any one time.

Alternate members shall have voting rights.

#### Consultants

6.5.2.4 Consultants shall be utilized as determined by the NFSC Chairman.

### Meeting Frequency

6.5.2.5 The NFSC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

### Quorum

6.5.2.6 A majority of the permanent voting committee members, or duly appointed alternates, which shall include the Chairman or the Vice Chairman and of which a minority are from the Nuclear Power Generation Department shall constitute a quorum for meetings of the committee. In the event both the Chairman and the Vice Chairman are absent, one of the permanent voting members will serve as Acting Chairman.

### Review

6.5.2.7 The following subjects shall be reported to and reviewed by the Committee insofar as they relate to matters of nuclear safety:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or licenses.

6.6

REPORTABLE OCCURENCE ACTION

6.6.1 The following actions shall be taken in the event of a REPORTABLE OCCURENCE:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the SNSC and submitted to the NFSC Chairman, and the Manager, Nuclear Power Generation Department.

6.7

SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(1) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Commission, the Manager, Nuclear Power Generation Department and to the NFSC Chairman immediately.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NFSC Chairman and the Manager, Nuclear Power Generation Department within 10 days of the violation.

6.8

PROCEDURES

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix A of Regulatory Guide 1.33 (November 1972) except as provided in 6.8.2 and 6.8.3, below.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and any changes to them shall be reviewed and approved for implementation in accordance with a written administrative control procedure approved by the Manager, Nuclear Power Generation Department, with the concurrence of the Station Nuclear Safety Committee and the Nuclear Facilities Safety Committee. The administrative control procedure required by this specification shall, as a minimum, require that:

- a. Each proposed procedure/procedure change involving safety related components and/or operation of same receives a pre-implementation review by the SNSC except in case of an emergency.
- b. Each proposed procedure/procedure change which renders or may render the Final Safety Analysis Report or subsequent safety analysis reports inaccurate and those which involve or may involve potential unreviewed safety questions are approved by the SNSC prior to implementation.
- c. The approval of the Nuclear Facilities Safety Committee shall be sought if, following its review, the Station Nuclear Safety Committee finds that the proposed procedure/procedure change either involves an unreviewed safety question or if it is in doubt as to whether or not an unreviewed safety question is involved.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 26 TO LICENSE NO. DPR-26 AND

AMENDMENT NO. 5 TO LICENSE NO. DPR-64

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

POWER AUTHORITY OF THE STATE OF NEW YORK

INDIAN POINT NUCLEAR GENERATING UNITS NOS. 2 AND 3

DOCKETS NOS. 50-247 AND 50-286

Introduction

By letters dated August 27, 1976 and November 9, 1976, the Consolidated Edison Company of New York, Inc. (Con Ed) requested amendments to the Technical Specifications appended to Facility Operating Licenses Nos. DPR-26 and DPR-64 for Indian Point Nuclear Generating Units Nos. 2 and 3, respectively. The proposed amendments involve planned organizational changes as follows: (1) the Plant Manager, instead of the Technical Engineering Director, would become Chairman of the Station Nuclear Safety Committee; (2) the Quality Assurance Engineer and the Security Supervisor would be added to the Station Nuclear Safety Committee; (3) reports generated by the Nuclear Safety Committee would be sent directly from the Safety Committee to the Manager, Nuclear Power Generation Department, and the Chairman of the Nuclear Facilities Safety Committee instead of being routed through the Plant Manager; (4) corrections would be made in the Unit No. 2 Technical Specifications for respiratory protection to update obsolete references; and (5) changes in Con Ed organization would be allowed without prior NRC approval with the changes reported to the NRC within 30 days.

Evaluation

During our review of the proposed Technical Specifications we determined that proposed section 6.2.3 and the related proposed changes in reporting requirements should be deleted as they would allow organizational changes prior to NRC authorization. We also determined that the Reactor Engineer should report directly to the Plant Manager to assure that the Plant Manager had direct control over the employee responsible for certain reactor operational parameters important to safety.

Changes were made in the organizational charts (figures 6.2.1 and 6.2.2) to more clearly reflect the fact that certain engineers and directors had staffs reporting to them. The composition of the Station Nuclear Safety Committee was modified to include the Manager, Nuclear Power Generation as a member. The highest level on-site manager would therefore be more directly informed of safety considerations and would also participate in such considerations by the committee.

The proposed technical specifications as modified provide improvements in administrative controls that are consistent with Regulatory Guide 1.33 "Quality Assurance Program Requirements," Regulatory Guide 1.8 "Personnel Selection and Training," and the Standard Technical Specifications. The proposed change in the membership of the Station Nuclear Safety Committee involves the Plant Manager (as Chairman) in direct participation in the considerations of that committee. As Committee Chairman, the Plant Manager will have more immediate and complete knowledge of the details of all safety considerations. The inclusion in the Committee of all on-site upper management staff, concerned with operation or maintenance of the plant, will assure that all significant safety issues from any on-site organization will be brought before the Committee.

The change in referenced regulations for respiratory protection for Unit No. 2 has no safety implications but simply updates the references to be consistent with the present chapter designations in the regulations and the Unit No. 3 specifications.

### Environmental Consideration

We have determined that these amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 4, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-247 AND 50-286

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

POWER AUTHORITY OF THE STATE OF NEW YORK

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued to Consolidated Edison Company of New York, Inc. (Con Ed), Amendment No. 26 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2, and has issued to Con Ed and the Power Authority of the State of New York, Amendment No. 5 to Facility Operating License No. DPR-64 for Indian Point Nuclear Generating Unit No. 3. These amendments revised Technical Specifications for operation of Indian Point Units Nos. 2 and 3 located in Buchanan, Westchester County, New York. The amendments are effective as of the date of issuance.

These amendments revise the Technical Specifications to change requirements for administrative controls at each unit. The changes principally pertain to assignments of key personnel within the plant organization.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

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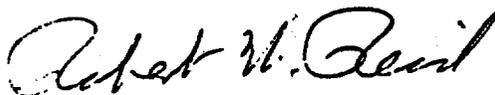
The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments transmitted by letters dated August 27, 1976 and November 9, 1976, (2) Amendment No. 26 to License No. DPR-26, (3) Amendment No. 5 to License No. DPR-64, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Hendrick Hudson Free Library, 31 Albany Post Road, Montrose, New York.

A copy of items (2), (3), and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 4th day of January 1977.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors