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Docket No.: 50-247

MAR. 30 1976

Consolidated Edison Company  
 of New York, Inc.  
 ATTN: Mr. William J. Cahill, Jr.  
 Vice President  
 4 Irving Place  
 New York, New York 10003

Gentlemen:

The Commission has issued the enclosed Amendment No. 19 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated August 25, 1975. As discussed with your staff, modifications have been made to your proposed changes to meet regulatory requirements.

The amendment revises the provisions of the Technical Specifications to clarify surveillance test requirements during plant conditions other than power operations.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Original Signed By

Robert W. Reid, Chief  
 Operating Reactors Branch #4  
 Division of Operating Reactors

Enclosures:

1. Amendment No. 19
2. Safety Evaluation
3. Federal Register Notice

cc w/enclosures: See next page

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Consolidated Edison Company

cc w/ enclosures:

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New York State Department of Commerce  
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Albany, New York 12210



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

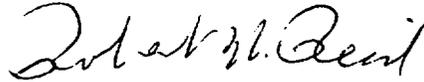
Amendment No. 19  
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) sworn to August 21, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: MAR. 30 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 19

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Replace the existing pages of the Technical Specifications listed below with the attached revised pages bearing the same numbers.

Changes on these pages are shown by marginal lines.

Pages

4.1-1

4.1-2

## SURVEILLANCE REQUIREMENTS

### 4.1 OPERATIONAL SAFETY REVIEW

#### Applicability

Applies to items directly related to safety limits and limiting conditions for operation.

#### Objective

To specify the minimum frequency and type of surveillance to be applied to plant equipment and conditions.

#### Specification

- a. Calibration, testing and checking of analog channels, and testing of logic channels shall be performed as specified in Table 4.1-1.
- b. Sampling and equipment tests shall be conducted as specified in Tables 4.1-2 and 4.1-3, respectively.
- c. Performance of any surveillance test outlined in these specifications is not immediately required if the plant condition is the same as the condition into which the plant would be placed by an unsatisfactory result of that test. Such tests will be performed before the plant is removed from the subject condition that has precluded the immediate need to run the test. If the test provisions require that a minimum higher system condition must first be established, the test will be performed promptly upon achieving this minimum condition. The following surveillance tests, however, must be performed without the above exception:

•Table 4.1-1	Items 3, 19, 25, and 28
•Table 4.1-2	Items 1, 2, and 10 thru 17
•Table 4.1-3	Items 2, 6, 7, 12, and 13

#### Basis

A surveillance test is intended to identify conditions in a plant that would lead to a degradation of reactor safety. Should a test reveal such a condition, the Technical Specifications require that either immediately, or after a specified period of time, the plant be placed in a condition which mitigates or eliminates the consequences of additional related casualties or accidents. If the plant is already in a condition which satisfies the failure criteria of the test, then plant safety is not compromised and performance of the test yields information that is not necessary to determine safety limits or limiting conditions for operation of the plant. The surveillance test need not be performed, therefore, as long as the plant remains in this condition. However, this surveillance test should be performed prior to removing the plant from the subject condition that has precluded the immediate need to run the

test. In the situation in which the test provisions specify that the test must be performed at some minimum system condition, this condition will first be achieved and the test will be performed promptly thereafter prior to proceeding to a higher system condition.

a. Check

Failures such as blown instrument fuses, defective indicators, faulted amplifiers which result in "upscale" or "downscale" indication can be easily recognized by simple observation of the functioning of an instrument or system. Furthermore, such failures are, in many cases, revealed by alarm action, and a Check supplements this type of built-in surveillance.

Based on experience in operation of both conventional and nuclear plant systems, the minimum Checking frequency of once per shift when the plant is in operation, is deemed adequate for reactor and steam system instrumentation.

b. Calibration

Calibrations are performed to ensure the presentation and acquisition of accurate information.

The nuclear flux (linear level) channels are calibrated daily against a heat balance standard to account for errors induced by changing rod patterns and core physics parameters.

Other channels are subject only to the "drift" errors induced within the instrumentation itself and, consequently, can tolerate longer intervals between calibration. Process system instrumentation errors induced by drift can be expected to remain within acceptable tolerances if recalibration is performed at intervals of each refueling shutdown.

Substantial calibration shifts within a channel (essentially a channel failure) will be revealed during routine checking and testing procedures.

Thus, minimum calibration frequencies of once-per-day for the nuclear flux (linear level) channels, and once each refueling shutdown for the process system channels is considered acceptable.

c. Testing

The minimum testing frequency for those instrument channels connected to the safety system is based on an average unsafe failure rate of

SAFETY EVALUATION BY THE OFFICE OF

NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 19 TO FACILITY LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY

OF NEW YORK, INCORPORATED

INDIAN POINT NUCLEAR GENERATING

UNIT NO. 2

DOCKET NO. 50-247

Introduction

By letter dated August 25, 1975, Consolidated Edison Company of New York, Inc. (Con Ed) proposed an amendment to the Technical Specifications appended to Facility Operating License No. DPR-26 for Indian Point Nuclear Generating Unit No. 2. The proposed amendment would revise the provisions of the Technical Specifications to clarify surveillance test requirements during plant conditions other than power operations.

Discussion

During cold shutdown and refueling operations many of the surveillance tests required by the present technical specifications are not necessary or meaningful. Other surveillance tests cannot be accomplished during the cold shutdown or refueling conditions. The proposed amendment will allow the unnecessary tests to be postponed or deleted during cold shutdown and refueling.

Evaluation

During our review of Con Ed's application, we determined that modifications in the proposed amendment were needed to assure that necessary testing requirements were not deleted during plant shutdown and refueling conditions. These modifications were made.

The proposed amendment as modified is a clarification and will allow the deletion of certain surveillance testing when the plant is already in the condition that satisfies the failure criteria of the test except for tests which must continue during cold shutdown or refueling operations. Deletion of these surveillance tests does not compromise plant safety because the plant is in the condition that

would be required by unsatisfactory test results. The proposed specification, as modified, also assures that appropriate surveillance testing is accomplished prior to a return to power operations. When power operation is required as a condition of the surveillance test, the proposed specification, as modified, specifies that the test be performed promptly after a return to power. The probability or consequences of accidents is not increased and there is no decrease in safety margins because this amendment is only to clarify surveillance requirements and all necessary surveillance tests continue to be required.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: **MAR. 30 1978**

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-247

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 19 to Facility Operating License No. DPR-26 issued to Consolidated Edison Company of New York, Inc. which revised Technical Specifications for operation of Indian Point Nuclear Generating Unit No. 2, located in Buchanan, Westchester County, New York. The amendment is effective as of its date of issuance.

This amendment revises the provisions of the Technical Specifications to clarify surveillance test requirements during plant conditions other than power operations.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

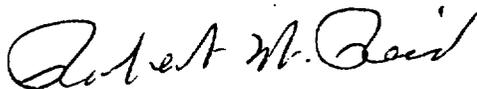
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For further details with respect to this action, see (1) the application for amendment submitted by letter dated August 25, 1975, (2) Amendment No. 19 to License No. DPR-26, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Hendrick Hudson Free Library, 31 Albany Post Road, Montrose, New York.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 30th day of March 1976.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Operating Reactors