

Docket No. 50-247

Consolidated Edison Company
of New York, Inc.
ATTN: Mr. William J. Cahill, Jr.
Vice President
4 Irving Place
New York, New York 10003

Change No. 3
License No. DPR-26

Gentlemen:

Your letter dated September 14, 1973 requested a change to the Technical Specifications incorporated as Appendix A to Facility Operating License DPR-26. This change was requested to correct a typographical error. Your request has been denoted as Change No. 3.

Paragraph 3.9.B.5 on page 3.9-5 of Section 3.9, Effluent Release - Radioactive Materials, specifies the maximum radioactivity to be contained in one waste gas decay tank. As correctly presented in the basis for the section (page 3.9-13), the maximum inventory is determined so as to limit the dose at the site boundary to below the limits of 10 CFR Part 20 for the postulated rupture of a waste gas decay tank releasing all of its radioactivity. The maximum amount of radioactivity to be permitted in one waste gas decay tank on this basis was correctly determined to be 16,500 Ci (equivalent Xe-133 Curies) for Unit 2 but was inadvertently typed incorrectly as 16,500 uCi (equivalent Xe-133 Curies). Therefore 16,500 Ci should be substituted for 16,500 uCi in paragraph 3.9.B.5 of page 3.9.5.

We have concluded therefore, that the proposed change does not involve significant hazard considerations and that there is reasonable assurance that the health and safety of the public will not be endangered.

Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specification change outlined in your letter and Attachment I dated

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SURNAME➤							
DATE➤							

Consolidated Edison Company
of New York, Inc.

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September 14, 1973 is hereby authorized. To effect this change, please replace page 3.9-5 of the Technical Specifications of Facility Operating License No. DPR-26 with the revised page 3.9-5 (designated as Change No. 3 on the bottom of the page).

Sincerely,

Donald J. Skovholt, Assistant Director
for Operating Reactors
Directorate of Licensing

Enclosure:
Revised page 3.9-5

cc:

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New York State Atomic Energy Council
99 Washington Avenue
Albany, New York

Angus Macbeth, Esquire
Natural Resources Defense Council, Inc.
36 West 44th Street
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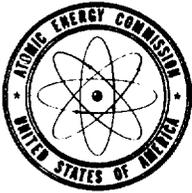
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DATE	1/22/74	1/24/74	1/24/74	1/24/74	



UNITED STATES
ATOMIC ENERGY COMMISSION
WASHINGTON, D.C. 20545

January 24, 1974

Docket No. 50-247

Consolidated Edison Company
of New York, Inc.
ATTN: Mr. William J. Cahill, Jr.
Vice President
4 Irving Place
New York, New York 10003

Change No. 3
License No. DPR-26

Gentlemen:

Your letter dated September 14, 1973 requested a change to the Technical Specifications incorporated as Appendix A to Facility Operating License DPR-26. This change was requested to correct a typographical error. Your request has been denoted as Change No. 3.

Paragraph 3.9.B.5 on page 3.9-5 of Section 3.9, Effluent Release - Radioactive Materials, specifies the maximum radioactivity to be contained in one waste gas decay tank. As correctly presented in the basis for the section (page 3.9-13), the maximum inventory is determined so as to limit the dose at the site boundary to below the limits of 10 CFR Part 20 for the postulated rupture of a waste gas decay tank releasing all of its radioactivity. The maximum amount of radioactivity to be permitted in one waste gas decay tank on this basis was correctly determined to be 16,500 Ci (equivalent Xe-133 Curies) for Unit 2 but was inadvertently typed incorrectly as 16,500 uCi (equivalent Xe-133 Curies). Therefore 16,500 Ci should be substituted for 16,500 uCi in paragraph 3.9.B.5 of page 3.9.5.

We have concluded therefore, that the proposed change does not involve significant hazard considerations and that there is reasonable assurance that the health and safety of the public will not be endangered.

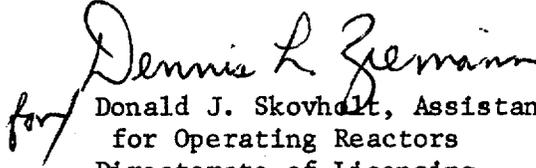
Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specification change outlined in your letter and Attachment I dated

Consolidated Edison Company
of New York, Inc.

-2-

September 14, 1973 is hereby authorized. To effect this change, please replace page 3.9-5 of the Technical Specifications of Facility Operating License No. DPR-26 with the revised page 3.9-5 (designated as Change No. 3 on the bottom of the page).

Sincerely,


for Donald J. Skovholt, Assistant Director
for Operating Reactors
Directorate of Licensing

Enclosure:
Revised page 3.9-5

cc:

Leonard M. Trosten, Esquire
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80 Centre Street
New York, New York 10013

gaseous release rates to within the values specified in 3.9.B.1 and 3.9.B.2 above shall be operable.

5. The maximum activity to be contained in one waste gas decay tank shall not exceed 11,400 Ci at Unit 1 and 16,500 Ci at Unit 2 (i.e. equivalent Xe-133 curies).
6. When it has been projected that the quarterly release of airborne radioactive effluents will exceed 12.5% of the limits specified in Specification 3.9.B.3, the appropriate equipment shall be used and procedures followed to significantly reduce airborne effluent activity.
7. When the release of radioactive gases exceeds 50% of the limits specified in Specification 3.9.B.3, the licensee shall report such as per Section 6.12.2.b.5 of the Technical Specifications.

Basis

Although it is expected that annual releases of liquid radioactive effluents will not result in exceeding a small fraction of the concentration limits of 10CFR20, Appendix B, Table II, Column 2, Specification 3.9.A.1 permits the flexibility of operation, compatible with considerations of health and safety, to assure that the public is provided a dependable source of power under unusual operating conditions which may result in releases higher than the design objective levels.

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limits (defined in the Technical Specifications) and with part length rod insertion restricted to 70 percent of full insertion. However, operation with part length rod insertion exceeding 70 percent during zero power or low power physics tests does not have the potential for exceeding the fuel design performance limits.

We have concluded therefore, that the proposed change does not involve significant hazard considerations not described or implicit in the Final Facility Description and Safety Analysis Report and that there is reasonable assurance that the health and safety of the public will not be endangered.

Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specification change outlined in your letter dated May 4, 1973 is hereby authorized. To effect this change, replace pages 3.10-1 and 3.10-2 of the Technical Specifications of Facility Operating License No. DPR-26 with the revised pages 3.10-1 (designated as Change No. 1 on the bottom of the page) and 3.10-2. enclosed.

Sincerely,

/s/

R. C. DeYoung, Assistant Director
for Pressurized Water Reactors
Directorate of Licensing

Enclosure:
Revised pages

ccs:
Listed on page 3

*5/16/73
Con. Ed. received
a copy of its
request on
5/16/73*

OFFICE ▶	PWR-2	PWR-2	DS/TR	CS/TR	AD/PWRs	OGC
SURNAME ▶	MMcCoy:nlg <i>MM</i>	KKniel <i>KK</i>	MDanenb <i>MD</i>	DRoss <i>DR</i>	RCDeYoung <i>RC</i>	MK <i>MK</i>
DATE ▶	5/15/73	5/15/73	5/15/73	5/15/73	5/16/73	5/16/73

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KKniel
MService
bcc: J.R. Buchanan, ORNL
Thomas B. Abernathy, DTIE

5/16/73
Docket No. 50-247

Consolidated Edison Company
of New York, Inc.
ATTN: Mr. Peter Zarakas
Assistant Vice President
4 Irving Place
New York, New York 10003

Change No. 1
License No. DPR-26

Gentlemen:

Your letter dated May 4, 1973, requested a change in the Technical Specifications attached as Appendix A to Amendment 2 of Facility Operating License No. DPR-26 authorizing operation of Indian Point Unit No. 2 for testing purposes at reactor core power levels not in excess of 50 percent of the rated power. This change was requested to permit physics testing and control rod exercises associated with initial tests and operations authorized by Amendment 2 of Facility Operating License No. DPR-26. Your request has been denoted as Change No. 1 to Amendment No. 2.

The initial testing program, including the part-length control rod exercises, for Indian Point Unit No. 2 is outlined in Section 13 of the Unit No. 2 Final Facility Description and Safety Analysis Report, which has been previously evaluated and found to be acceptable by us in connection with our review of Indian Point Unit No. 2. Part of the testing program involves withdrawal and insertion of selected control rods and part-length rods in connection with rod worth measurements and calibration of the ex-core detectors. The purpose of this change is to provide consistency with the exceptions permitted for full length control rods as given in Technical Specifications 3.10.1.2 and 3.10.1.4.

The basis for limiting part length rod insertion to 70 percent is to eliminate certain adverse power shapes that could result in high values of the nuclear heat flux hot channel factor F_N . In our evaluation of the design fuel performance limits, we concluded that the maximum allowable nuclear heat flux hot channel factor to meet these limits during full power operation would not be exceeded if the axial offset was restricted within appropriate

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PDR

ccs:

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Sydney Kingsley, Esquire
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