

October 17, 1978

Docket No. 50-247

Docket No. 50-247

Consolidated Edison Company
of New York, Inc.
ATTN: Mr. William J. Cahill, Jr.
Vice President
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Docket 50-247

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Gentlemen:

The Commission has issued the enclosed Amendment No. to facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. This amendment consists of changes to the license and Technical Specifications in response to your application transmitted by letter dated March 21, 1978.

The amendment changes the license to provide generalized provisions for sealed radioactive sources and changes the Technical Specifications to provide for standard surveillance requirements for these sources.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

- 1. Amendment No. to DPR-26
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures:
See next page

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Consolidated Edison Company of
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October 17, 1978

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 41
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated March 21, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. DPR-26, is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the Attachment to this license amendment:

A. Delete existing paragraphs 2.B.(2) and 2.B.(3), renumber existing paragraph 2.B.(4) as 2.B.(5), and insert new paragraphs 2.B.(2), 2.B.(3) and 2.B.(4) as follows:

- 2.B. (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Facility Description and Safety Analysis Report, as supplemented and amended, and as described in the Commission's authorization for Amendment No. 14 to this license.
- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;

B. Revise paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 41, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 17, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 41
FACILITY OPERATING LICENSE NO. DPR-26
DOCKET 50-247

Revise Appendix A as follows:

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4.15 RADIOACTIVE MATERIALS SURVEILLANCE

Applicability

Applies to the surveillance of sealed special nuclear, source and byproduct material sources.

Objective

To assure that leakage from byproduct, source, and special nuclear radioactive material sources does not exceed allowable limits.

Specification

- A. Tests for leakage and/or contamination shall be performed as follows:
1. Each sealed source, except startup sources and fission detectors, containing radioactive material, other than Hydrogen-3, with a half life greater than thirty days and in any form other than gas shall be tested for leakage and/or contamination at six month intervals.
 2. The periodic leak test required does not apply to sources that are stored and not being used. These sources shall be tested for leakage prior to any use or transfer to another user unless they have been leak tested within six months prior to the date of use or transfer. In the absence of a certificate from a transferor indicating that a test has been made within six months prior to the transfer, sealed sources shall not be put into use until tested.
 3. Primary startup sources and fission detectors shall be leak tested prior to being subjected to core flux and following repair or maintenance to the source.

- B. Sealed sources are exempt from Specification 4.15.A when the source contains 100 microcuries or less of beta and/or gamma emitting material or 5 microcuries or less of alpha emitting material.
- C. The leakage test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. If the test reveals the presence of 0.005 microcurie or more of removable contamination, the sealed source shall immediately be withdrawn from use and either decontaminated and repaired, or be disposed of in accordance with Commission regulations.
- D. A Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2.e within 30 days if source leakage tests reveal the presence of \geq 0.005 microcuries of removable contamination.

Basis

The objective of this specification is to assure that leakage from byproduct, source, and special nuclear radioactive material sources does not exceed the allowable limits specified in the Code of Federal Regulations.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Region I Office of Inspection and Enforcement within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Each containment integrated leak rate test shall be the subject of a summary technical report including results of the local leak rate test since the last report. The report shall include analyses and interpretations of the results which demonstrate compliance in meeting the leak rate limits specified in the Technical Specifications.
- b. A report covering the X-Y xenon stability tests within three months upon completion of the tests.
- c. To provide the Commission with added verifications of the safety and reliability of the pre-pressurized Zircaloy-clad nuclear fuel, a limited program of non-destructive fuel inspections will be conducted. The program shall consist of a visual inspection (e.g., underwater TV, periscope, or other) of the two lead burnup assemblies in each region during the first, second, and third refueling shutdowns. Any condition observed by this inspection which would lead to unacceptable fuel performance may be the object of an expanded surveillance effort. If another domestic plant which contains pre-pressurized fuel of a similar design reaches fuel exposures equal to or greater than at Indian Point Unit No. 2, and if a limited inspection program is or has been performed there, then the program may not have to be performed at Indian Point Unit No. 2. However, such action requires approval of the Nuclear Regulatory Commission. The results of these inspections will be reported to the Nuclear Regulatory Commission.
- d. Inoperable fire protection and detection equipment (Specification 3.13).
- e. Sealed source leakage in excess of limits (Specification 4.15).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

Introduction

By letter dated December 16, 1974 we requested that Consolidated Edison Company of New York (the licensee) propose a license amendment with changes to the Technical Specifications to incorporate generalized provisions for possession and use of byproduct, source and special nuclear materials. The license listed each item separately, thereby requiring a license amendment for almost any change in licensed materials.

By letter dated March 21, 1978 the licensee requested a license amendment and changes to the Technical Specifications in response to our request.

Evaluation

The proposed license amendment replaces the detailed list with more generalized provisions for possession and use of sealed byproduct, source and special nuclear material sources. The proposed change in the Technical Specifications assures that the leakage from these sealed sources does not exceed the allowable limits specified in the Code of Federal Regulations.

It should be noted that the proposed amendment relates only to sealed sources and does not involve an increase in the amount of special nuclear material as reactor fuel.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have

further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that:

- (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration,
- (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
- (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 17, 1978