

NOVEMBER 6 1978

Docket No. 50-247

Consolidated Edison Company
of New York, Inc.
ATTN: Mr. William J. Cahill, Jr.
Vice President
4 Irving Place
New York, New York 10003

DISTRIBUTION

Docket Files 50-247

Local PDR	W. Pasciak
NRC PDR	B. Harless
ORB 1 RF	B. Grimes
V. Stello	M. Donenfeld
D. Eisenhut	ACRS (16)
C. Parrish	OPA (Clare Miles)
L. Olshan	R. Diggs
OELD	J. Carter
OI&E (5)	TERA
B. Jones (8)	J. R. Buchanan,
B. Scharf (10)	
J. McGough	
J. Saltzman, AIG	
C. Hebron	
R. Ballard	

Gentlemen:

By our letter dated October 17, 1978, we transmitted to you Amendment No. 41 to Facility License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. This October 17, 1978 amendment should have been issued as Amendment No. 42 instead of Amendment No. 41. To correct this error we are herewith reissuing the October 17 amendment in its entirety.

Sincerely,

~~Original signature by~~

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosure:
Corrected October 17, 1978
amendment

*Construct
cep*

8111070227 781106
PDR ADOCK 05000247
P PDR

HS

OFFICE >	DOR:ORB#1/LA	DOR:ORB#1/PM	DOR:ORB#1/C			
SURNAME >	CParrish:jpf	LOlshan	ASchwencer			
DATE >	11/3/78	" 13/78	11/10/78			



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 6, 1978

Docket No. 50-247

Consolidated Edison Company
of New York, Inc.
ATTN: Mr. William J. Cahill, Jr.
Vice President
4 Irving Place
New York, New York 10003

Gentlemen:

By our letter dated October 17, 1978, we transmitted to you Amendment No. 41 to Facility License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. This October 17, 1978 amendment should have been issued as Amendment No. 42 instead of Amendment No. 41. To correct this error we are herewith reissuing the October 17 amendment in its entirety.

Sincerely,

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosure:
Corrected October 17, 1978
amendment

Consolidated Edison Company of
New York, Inc.

November 6, 1978

- 2 -

cc: White Plains Public Library
100 Martine Avenue
White Plains, New York 10601

Joseph D. Block, Esq.
Executive Vice President -
Administration
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Edward J. Sack, Esq.
Law Department
Consolidated Edison Company
of New York, Inc.
4 Irving Place
New York, New York 10003

Anthony Z. Roisman
Natural Resources Defense Council
917 - 15th Street, NW
Washington, D.C. 20005

Paul S. Shemin, Esquire
Assistant Attorney General
State of New York
Department of Law
Two World Trade Center
New York, New York 10047

Sarah Chasis, Esquire
Natural Resources Defense Council
122 East 42nd Street
New York, New York 10017

Carl R. D'Alvia, Esquire
Attorney for the Village of
Buchanan, New York
395 South Riverside Avenue
Croton-on-Hudson, New York 10520

Jeffrey C. Cohen, Esquire
New York State Energy Office
Swan Street Building
CORE 1- Second Floor
Empire State Plaza
Albany, New York 12223

Honorable George Begany
Mayor, Village of Buchanan
188 Westchester Avenue
Buchanan, New York 10511

Director, Technical Development
Programs
State of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Chief, Energy Systems
Analyses Branch (AW-459)
Office of Radiation Programs
U.S. Environmental Protection Agency
Room 645, East Tower
401 M Street, SW
Washington, D.C. 20460

U.S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated March 21, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-26, is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the Attachment to this license amendment:

A. Delete existing paragraphs 2.B.(2) and 2.B.(3), renumber existing paragraph 2.B.(4) as 2.B.(5), and insert new paragraphs 2.B.(2), 2.B.(3) and 2.B.(4) as follows:

2.B. (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Facility Description and Safety Analysis Report, as supplemented and amended, and as described in the Commission's authorization for Amendment No. 14 to this license.

(3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any by-product, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;

B. Revise paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 42, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 17, 1978

ATTACHMENT TO LICENSE AMENDMENT NO.42
FACILITY OPERATING LICENSE NO. DPR-26
DOCKET 50-247

Revise Appendix A as follows:

Remove Pages

ii

6.19

Insert Pages

ii

4.15.1

4.15.2

6.19

TABLE OF CONTENTS (Continued)

<u>Section</u>	<u>Title</u>	<u>Page</u>
3.10	Control Rod and Power Distribution Limits	3.10-1
	Shutdown Reactivity	3.10-1
	Power Distribution Limits	3.10-1
	Quadrant Power Tilt Limits	3.10-4
	Rod Insertion Limits	3.10-5
	Rod Misalignment Limitations	3.10-6
	Inoperable Rod Position Indicator Channels	3.10-6
	Inoperable Rod Limitations	3.10-7
	Rod Drop Time	3.10-7
	Rod Position Monitor	3.10-7
	Quadrant Power Tilt Monitor	3.10-7
	Notification	3.10-8
3.11	Movable In-Core Instrumentation	3.11-1
3.12	Shock Suppressors (Snubbers)	3.12-1
3.13	Fire Protection and Detection Systems	3.13-1
4	Surveillance Requirements	4.1-1
4.1	Operational Safety Review	4.1-1
4.2	Primary System Surveillance	4.2-1
4.3	Reactor Coolant System Integrity Testing	4.3-1
4.4	Containment Tests	4.4-1
	Integrated Leakage Rate Test - Pre-Operational	4.4-1
	Integrated Leakage Rate Test - Post-Operational	4.4-2
	Report of Test Results	4.4-4
	Continuous Leak Detection Testing via the Containment Penetration and Weld Channel Pressurization System	4.4-4
	Corrective Action	4.4-4
	Isolation Valve Tests	4.4-4
	Residual Heat Removal Systems	4.4-5
	Annual Inspection	4.4-6
	Containment Modification	4.4-6
4.5	Engineered Safety Features	4.5-1
	Safety Injection System	4.5-1
	Containment Spray System	4.5-2
	Hydrogen Recombiner System	4.5-2
	Component Tests	4.5-3
4.6	Emergency Power System Periodic Tests	4.6-1
	Diesel Generators	4.6-1
	Diesel Fuel Tanks	4.6-2
	Station Batteries	4.6-2
4.7	Main Steam Stop Valves	4.7-1
4.8	Auxiliary Feedwater System	4.8-1
4.9	Reactivity Anomalies	4.9-1
4.10	DELETED	
4.11	DELETED	
4.12	Shock Suppressors (Snubbers)	4.12-1
4.13	Steam Generator Tube Inservice Surveillance	4.13-1
	Inspection Requirements	4.13-1
	Corrective Measures	4.13-4
	Reports	4.13-4
4.14	Fire Protection and Detection Systems	4.14-1
4.15	Radioactive Materials Surveillance	4.15-1

4.15 RADIOACTIVE MATERIALS SURVEILLANCE

Applicability

Applies to the surveillance of sealed special nuclear, source and byproduct material sources.

Objective

To assure that leakage from byproduct, source, and special nuclear radioactive material sources does not exceed allowable limits.

Specification

- A. Tests for leakage and/or contamination shall be performed as follows:
1. Each sealed source, except startup sources and fission detectors, containing radioactive material, other than Hydrogen-3, with a half life greater than thirty days and in any form other than gas shall be tested for leakage and/or contamination at six month intervals.
 2. The periodic leak test required does not apply to sources that are stored and not being used. These sources shall be tested for leakage prior to any use or transfer to another user unless they have been leak tested within six months prior to the date of use or transfer. In the absence of a certificate from a transferor indicating that a test has been made within six months prior to the transfer, sealed sources shall not be put into use until tested.
 3. Primary startup sources and fission detectors shall be leak tested prior to being subjected to core flux and following repair or maintenance to the source.

- B. Sealed sources are exempt from Specification 4.15.A when the source contains 100 microcuries or less of beta and/or gamma emitting material or 5 microcuries or less of alpha emitting material.
- C. The leakage test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. If the test reveals the presence of 0.005 microcurie or more of removable contamination, the sealed source shall immediately be withdrawn from use and either decontaminated and repaired, or be disposed of in accordance with Commission regulations.
- D. A Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2.e within 30 days if source leakage tests reveal the presence of \geq 0.005 microcuries of removable contamination.

Basis

The objective of this specification is to assure that leakage from byproduct, source, and special nuclear radioactive material sources does not exceed the allowable limits specified in the Code of Federal Regulations.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Region I Office of Inspection and Enforcement within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Each containment integrated leak rate test shall be the subject of a summary technical report including results of the local leak rate test since the last report. The report shall include analyses and interpretations of the results which demonstrate compliance in meeting the leak rate limits specified in the Technical Specifications.
- b. A report covering the X-Y xenon stability tests within three months upon completion of the tests.
- c. To provide the Commission with added verifications of the safety and reliability of the pre-pressurized Zircaloy-clad nuclear fuel, a limited program of non-destructive fuel inspections will be conducted. The program shall consist of a visual inspection (e.g., underwater TV, periscope, or other) of the two lead burnup assemblies in each region during the first, second, and third refueling shutdowns. Any condition observed by this inspection which would lead to unacceptable fuel performance may be the object of an expanded surveillance effort. If another domestic plant which contains pre-pressurized fuel of a similar design reaches fuel exposures equal to or greater than at Indian Point Unit No. 2, and if a limited inspection program is or has been performed there, then the program may not have to be performed at Indian Point Unit No. 2. However, such action requires approval of the Nuclear Regulatory Commission. The results of these inspections will be reported to the Nuclear Regulatory Commission.
- d. Inoperable fire protection and detection equipment (Specification 3.13).
- e. Sealed source leakage in excess of limits (Specification 4.15).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 42 TO FACILITY OPERATING LICENSE NO. DPR-26
CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

Introduction

By letter dated December 16, 1974 we requested that Consolidated Edison Company of New York (the licensee) propose a license amendment with changes to the Technical Specifications to incorporate generalized provisions for possession and use of byproduct, source and special nuclear materials. The license listed each item separately, thereby requiring a license amendment for almost any change in licensed materials.

By letter dated March 21, 1978 the licensee requested a license amendment and changes to the Technical Specifications in response to our request.

Evaluation

The proposed license amendment replaces the detailed list with more generalized provisions for possession and use of sealed byproduct, source and special nuclear material sources. The proposed change in the Technical Specifications assures that the leakage from these sealed sources does not exceed the allowable limits specified in the Code of Federal Regulations.

It should be noted that the proposed amendment relates only to sealed sources and does not involve an increase in the amount of special nuclear material as reactor fuel.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have

DUPE

~~781150182~~

further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that:

- (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration,
- (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
- (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 17, 1978