

January 3, 2001

Mr. A. Alan Blind
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, NY 10511

SUBJECT: COMPLETION OF LICENSING ACTION FOR GENERIC LETTER 97-05 -
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2 (TAC NO. MA0468)

Dear Mr. Blind:

On January 29 1998, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter (GL) 97-05, "Steam Generator Tube Inspection Techniques," to all holders of operating licenses for pressurized-water reactors. The NRC issued GL 97-05 to determine whether it was the licensee's practice to leave steam generator tubes with indications in service based on sizing. If so, the NRC staff requested a written report that included, for each type of indication, a description of the associated nondestructive examination method being used and the technical basis for the acceptability of the technique used.

In a letter dated March 17, 1998, as supplemented on October 28, 1998, and October 1, 1999, the Consolidated Edison Company of New York, Inc. (ConEd) provided its response to GL 97-05 for the Indian Point Nuclear Generating Unit No. 2 (IP2). In these letters, ConEd included a description and technical basis for the steam generator tube inspection techniques. After reviewing the information in these letters, the NRC staff found that ConEd's inspection techniques in sizing pit indications in steam generator tubes may not be acceptable because the technique may not be qualified in accordance with the requirements of Appendix B to 10 CFR Part 50.

On September 8, 2000, the NRC staff requested that ConEd provide additional information to clarify its position on the use of eddy current techniques to size pit indications and the criteria for repair of tubes with pit indications. In a letter dated November 7, 2000, ConEd confirmed that during the 2000 steam generator inspection, tubes that contained pits were plugged on detection. However, ConEd stated that it would continue to size pits for information only.

On August 8, 2000, ConEd informed the NRC that the existing Westinghouse model 44 steam generators would be replaced with Westinghouse model 44F steam generators before restart of IP2. In support of the replacement, ConEd performed a pre-service inspection of the replacement steam generators and found no degradation. In its November 7, 2000, letter, ConEd stated that it had implemented Station Administrative Order-180, "Administrative Steam Generator Program Plan," which describes its program for implementation of the industry guidelines for steam generator inspection and assessment. These guidelines detail steam generator tube inspection procedures, eddy current technique qualification, and reporting

A. Blind

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requirements. ConEd also stated that it will document the methods used to detect and size steam generator tube degradation each cycle in a degradation assessment. Further, ConEd committed that before leaving degraded tubes in service, it will submit to the NRC the applicable plugging criteria, details of the inspection method, and technical bases.

The NRC staff has reviewed the information provided by ConEd and finds that the steam generator tube inspection techniques used at IP2 comply with the current licensing basis and GL 97-05. Therefore, TAC No. MA0468 is closed.

If you have any questions regarding this matter, please contact me at (301) 415-1457.

Sincerely,

/RA/

Patrick D. Milano, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-247

cc: See next page

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Indian Point Nuclear Generating Station
Units 2

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, NY 10511

Mr. F. William Valentino, President
New York State Energy, Research,
and Development Authority
Corporate Plaza West
286 Washington Ave. Extension
Albany, NY 12203-6399

Mr. John McCann
Manager of Nuclear Safety and
Licensing
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, NY 10511

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P.O. Box 38
Buchanan, NY 10511

Mr. Brent L. Brandenburg
Assistant General Counsel
Consolidated Edison Company
of New York, Inc.
4 Irving Place - 1822
New York, NY 10003

David Lochbaum
Nuclear Safety Engineer
Union of Concerned Scientists
1707 H Street, NW., Suite 600
Washington, DC 20006

Edward Smeloff
Pace University School of Law
The Energy Project
78 North Broadway
White Plains, NY 10603

Charles Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, NY 10271

Ms. Charlene D. Faison, Director
Nuclear Licensing
Power Authority of the State
of New York
123 Main Street
White Plains, NY 10601

Mr. Thomas Rose
Secretary - NFSC
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, NY 10511

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Paul Eddy
New York State Department of
Public Service
3 Empire State Plaza, 10th Floor
Albany, NY 12223

Jim Riccio
Public Citizen's Critical Mass Energy Project
215 Pennsylvania Ave., SE
Washington, DC 20003

Michael Mariotte
Nuclear Information & Resources Service
1424 16th Street, NW, Suite 404
Washington, DC 20036