

September 25, 1989

Docket No. 50-412

Mr. J. D. Sieber, Vice President  
Nuclear Group  
Duquesne Light Company  
Post Office Box 4  
Shippingport, Pennsylvania 15077

Dear Mr. Sieber:

SUBJECT: BEAVER VALLEY UNIT 2 - ISSUANCE OF AMENDMENT (TAC NO. 74111)

The Commission has issued the enclosed Amendment No. 21 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit 2, in response to your application dated July 27, 1989.

The amendment revises certain visual inspection criteria for snubbers and the service life monitoring requirements.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

*Signed by*

Peter S. Tam, Senior Project Manager  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 21 to NPF-73
- 2. Safety Evaluation

cc w/enclosures:  
See next page

[BV AMEND 74111]

LA:PDI-4  
SNOPPTS  
09/11/89

PM:PDI-4  
PTam:blb  
09/11/89

D:PDI-4  
JStolz  
09/15/89

OGC [initials]  
11/15/89  
OGC comments on the SE have been incorporated.

*EP 1*

*PST 9/15/89*

*DF 11*

DATED: September 25, 1989

AMENDMENT NO. 21 TO FACILITY OPERATING LICENSE NO. NPF-73 - BEAVER VALLEY, UNIT 2

Docket File

NRC & Local PDRs

Plant File

SVarga, 14/E/4

BBoger, 14/A/2

JStolz

SNorris

PTam

OGC (for information only)

DHagan, 3302 MNBB

EJordan, 3302 MNBB

BGrimes, 9/A/2

TMeek (4)

Wanda Jones

J. Calvo

ACRS (10)

GPA/PA

OC/LFMB

cc: Plant Service list

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Duquesne Light Company

Beaver Valley Power Station  
Units 1 & 2

cc:

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Mr. J. D. Sieber, Vice President  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 21  
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated July 27, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

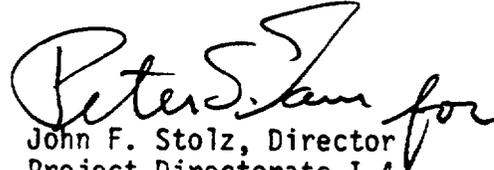
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 21, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DLCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 25, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. - 21

FACILITY OPERATING LICENSE NO. - NPF-73

DOCKET NO. - 50-412

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 7-25

B3/4 7-5

Insert

3/4 7-25

B3/4 7-5

## PLANT SYSTEMS

### SURVEILLANCE REQUIREMENTS (CONTINUED)

The snubbers may be categorized into two groups: those accessible and those inaccessible during reactor operation. Each group may be inspected independently in accordance with the above schedule.

#### b. Visual Inspection Criteria

Visual inspections shall verify (1) that there are no visible indications of damage or impaired OPERABILITY, (2) attachments to the foundation or supporting structure are secure, and (3) in those locations where snubber movement can be manually induced without disconnecting the snubber, that the snubber has freedom of movement and is not frozen up. Snubbers which appear inoperable as a result of visual inspections may be determined OPERABLE for the purpose of establishing the next visual inspection interval, providing that (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers that may be generically susceptible; or (2) the affected snubber is functionally tested in the as-found condition and determined OPERABLE per Specification 4.7.12.d. However, when a fluid port of a hydraulic snubber is found to be uncovered, the snubber shall be determined inoperable and cannot be determined OPERABLE via functional testing for the purpose of establishing the next visual inspection interval.

Snubbers which have been determined to be inoperable as a result of unexpected transients, isolated damage, or other random events, and cannot be proven operable by functional testing for the same reasons, shall not be counted in determining the next visual inspection period when the provision in 4.7.12.c (that failures are subject to an engineering evaluation of component structural integrity) has been met and equipment has been restored to an operable state via repair and/or replacement as necessary.

#### c. Functional Tests

At least once per 18 months during shutdown, a representative sample (of at least 10%) of the total of each type of snubber in use in the plant shall be functionally tested either in place or in a bench test. For Functional Testing type of snubber shall mean a group or combination of groups by load size and kind (i.e., hydraulic or mechanical) or any other combination of load size and kind. For each snubber that does not meet the functional test acceptance criteria of Specification 4.7.12.d, an additional 10% shall be functionally tested.

## 3/4.7 PLANT SYSTEMS

### BASES

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#### 3/4.7.9 SEALED SOURCE CONTAMINATION

The limitations on sealed source removable contamination ensure that the total body or individual organ irradiation does not exceed allowable limits in the event of ingestion or inhalation of the source material. The limitations on removable contamination for sources requiring leak testing, including alpha emitters, is based on 10 CFR 70.39(c) limits for plutonium. Leakage of sources excluded from the requirements of this specification represent less than one maximum permissible body burden for total body irradiation if the source material is inhaled or ingested.

Sealed sources are classified into three groups according to their use, with surveillance requirements commensurate with the probability of damage to a source in that group. Those sources which are frequently handled are required to be tested more often than those which are not. Sealed sources which are continuously enclosed within a shielded mechanism (i.e., sealed sources within radiation monitoring or boron measuring devices) are considered to be stored and need not be tested unless they are removed from the shielded mechanism.

#### 3/4.7.10 and 3/4.7.11 RESIDUAL HEAT REMOVAL SYSTEM (RHR)

Deleted

#### 3/4.7.12 SNUBBERS

All snubbers are required OPERABLE to ensure that the structural integrity of the reactor coolant system and all other safety-related systems is maintained during and following a seismic or other similar event initiating dynamic loads. Snubbers excluded from this inspection program are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

The visual inspection frequency is based upon maintaining a constant level of snubber protection to systems. Therefore, the required inspection interval varies inversely with the observed snubber failures and is determined by the number of inoperable snubbers found during an inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection.

When the cause of the rejection of a snubber is clearly established and remedied for that snubber and for any other snubbers that may be generically susceptible, or verified OPERABLE by inservice functional testing, that snubber may be exempted from being counted as inoperable. Generically, susceptible snubbers are those which are of a specific make or model and have the same



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 21 TO FACILITY OPERATING LICENSE NO. NPF-73

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 2

DOCKET NO. 50-412

INTRODUCTION

By letter dated July 27, 1989, Duquesne Light Company (the licensee, acting as agent for the above utilities) submitted a request to amend the Beaver Valley 2 Technical Specifications to revise certain snubber inspection criteria. This application is identical to parts of an application submitted under Unit 1's docket dated November 12, 1986. The Unit 1 application was approved by Amendment No. 135.

DISCUSSION AND EVALUATION

Section 4.7.12.b, "Visual Inspection Acceptance Criteria"

The change amounts to changing the word "and" to "or" in the middle of the paragraph. The change allows a snubber which appears inoperable to be determined operable provided that either the cause of rejection is remedied for that snubber and other snubbers that may be generically susceptible, or the affected snubber passes the functional testing criteria. The "or" provision constitutes a relaxation from the former requirement, which required both conditions ("and") be satisfied. The licensee defended this relaxation by using examples. We agree with the licensee's argument that the "or" requirement is more reasonable, would reduce occupational radiation exposure and would not significantly decrease assurance of snubber operability. This change was previously approved for Beaver Valley Unit 1 (Amendment No. 135) and is acceptable.

A new paragraph is added which permits an inoperable snubber that cannot be determined operable by functional testing to be declared operable for the purpose of establishing a new inspection interval, if it can be determined that the snubber was rendered inoperable as a result of unexpected transients, isolated damage or other random events. Examples of events which would be considered random or isolated include an object inadvertently dropped on a snubber or damage due to work in progress. An engineering evaluation of component structural integrity would still be performed after each failure.

If it can be determined that a snubber was rendered inoperable as a result of unexpected transients, isolated damage or other random events, similar failures would not be anticipated. Therefore, additional inspections should not be required to verify overall snubber operability since the cause is external. (This change is identical to Amendment No. 72 issued to North Anna on November 21, 1985.) This change was previously approved for Beaver Valley Unit 1 and is acceptable.

Bases Section 3/4.7.12, "Snubbers"

Appropriate paragraphs have been revised to reflect the above Technical Specification changes.

#### ENVIRONMENTAL CONSIDERATION

This amendment changes requirements with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The staff has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 25, 1989

Principal Contributor:

Peter S. Tam