



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 7, 1994

Mr. J. E. Cross  
Senior Vice President and  
Chief Nuclear Officer  
Nuclear Power Division  
Duquesne Light Company  
Post Office Box 4  
Shippingport, PA 15077

SUBJECT: ADMINISTRATIVE ERRORS IN AMENDMENT NOS. 181 AND 61, BEAVER VALLEY  
POWER STATION, UNIT NOS. 1 AND 2 (TAC NOS. M88613 AND M88614)

Dear Mr. Cross:

The purpose of this letter is to correct administrative errors in Amendment Nos. 181 and 61 for the Beaver Valley Power Station, Unit Nos. 1 and 2.

In a teleconference on October 25, 1994, Duquesne Light Company (DLC), reported errors found in Amendment No. 181 for Beaver Valley Power Station, Unit 1, and Amendment No. 61 for Beaver Valley Power Station, Unit 2. The U.S. Nuclear Regulatory Commission (NRC) issued Amendment Nos. 181 and 61 by letter dated August 8, 1994, in response to a DLC submittal dated October 22, 1993. The October 22, 1993, DLC submittal included three attachments; the proposed marked-up Technical Specifications (TSs) (Attachments A-1 and A-2), the safety analysis (Attachment B), and the proposed final-typed replacement TS pages (Attachments C-1 and C-2). DLC explained that the errors first appeared in the proposed replacement TS pages, Attachments C-1 and C-2, and did not appear in the marked-up TSs, Attachments A-1 and A-2. Enclosure 1 lists the errors found in Amendment Nos. 181 and 61.

The NRC staff has reviewed the errors DLC reported in the October 25, 1994, teleconference. Because the NRC staff based its safety evaluation (SE) upon the marked-up TS pages in Attachments A-1 and A-2 (which did not include the errors), no change to the SE is necessary. Therefore, we are issuing the corrected TS pages to replace the pages with errors. Enclosure 2 contains a list of the pages for replacement, followed by the replacement pages.

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During the teleconference, DLC staff explained that errors occurred in transcription from the marked-up TS pages to the typed TS pages. To ensure that similar transcription errors do not occur in the future, it is suggested that DLC consider whether improvement is needed in the quality check system now in place.

By this letter, we are closing TAC Nos. M88613 and M88614.

Sincerely,

Original signed by  
Gordon E. Edison, Senior Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-334  
and 50-412

Enclosures: 1. List of Errors  
2. Correction Pages

cc w/encls: See next page

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During the teleconference, DLC staff explained that errors occurred in transcription from the marked-up TS pages to the typed TS pages. To ensure that similar transcription errors do not occur in the future, it is suggested that DLC consider whether improvement is needed in the quality check system now in place.

By this letter, we are closing TAC Nos. M88613 and M88614.

Sincerely,



Gordon E. Edison, Senior Project Manager  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-334  
and 50-412

Enclosures: 1. List of Errors  
2. Correction Pages

cc w/encls: See next page

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Duquesne Light Company

Beaver Valley Power Station  
Units 1 & 2

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ADMINISTRATIVE ERRORS IN AMENDMENT NOS. 181 AND 61

There were four errors reported by DLC in the October 25, 1994, teleconference, one involving Unit 1 and the remaining three involving Unit 2.

1. Unit 1 (Amendment No. 181) - Page 3/4 3-4

Functional Unit: 21. Reactor Trip Breakers

Description of error: APPLICABLE MODE 3 is in the same row as ACTION 40.

Correction: APPLICABLE MODE 3 changed to be in the same row as ACTION 39.

Functional Unit: 22. Automatic Trip Logic

Description of error: APPLICABLE MODE 3 is in the same row as ACTION 1.

Correction: APPLICABLE MODE 3 changed to be in the same row as ACTION 39.

2. Unit 2 (Amendment No. 61) - Page 3/4 3-4

Functional Unit: 21. Reactor Trip Breakers

Description of error: APPLICABLE MODE 3 is in the same row as ACTION 40.

Correction: APPLICABLE MODE 3 changed to be in the same row as ACTION 39.

Functional Unit: 22. Automatic Trip Logic

Description of error: APPLICABLE MODE 3 is in the same row as ACTION 1.

Correction: APPLICABLE MODE 3 changed to be in the same row as ACTION 39.

3. Unit 2 (Amendment No. 61) - Page 3/4 3-4

Functional Unit: 23. Reactor Trip System Interlocks

a. Intermediate Range Neutron Flux, P-6

Description of error: MINIMUM CHANNELS OPERABLE reads 1.

Correction: MINIMUM CHANNELS OPERABLE changed to read 2.

4. Unit 2 (Amendment No. 61) - Page 3/4 3-18

Functional Unit: 4. Steam Line Isolation

Description of error: Part b. reads "Automatic Actuation Logic".

Correction: Part b. changed to read "Automatic Actuation Logic and Actuation Relays".

CORRECTIONS TO LICENSE AMENDMENT NOS. 181 AND 61  
FACILITY OPERATING LICENSE NOS. DPR-66 AND NPF-73  
DOCKET NOS. 50-334 AND 50-412

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

Unit 1, 3/4 3-4  
Unit 2, 3/4 3-4  
Unit 2, 3/4 3-18

Insert

Unit 1, 3/4 3-4  
Unit 2, 3/4 3-4  
Unit 2, 3/4 3-18

TABLE 3.3-1 (Continued)

DPR-66

REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
18. Turbine Trip (Above P-9)					
A. Auto Stop Oil Pressure	3	2	2	1	7
B. Turbine Stop Valve Closure	4	4	4	1	8
19. Safety Injection Input from ESF	2	1	2	1, 2	1
20. Reactor Coolant Pump Breaker Position Trip (Above P-7)	1/breaker	2	1/breaker per operating loop	1	11
21. Reactor Trip Breakers	2	1	2	1, 2	40
	2	1	2	3 <sup>(3)</sup> , 4 <sup>(3)</sup> , 5 <sup>(3)</sup>	39
22. Automatic Trip Logic	2	1	2	1, 2	1
	2	1	2	3 <sup>(3)</sup> , 4 <sup>(3)</sup> , 5 <sup>(3)</sup>	39
23. Reactor Trip System Interlocks					
A. Intermediate Range Neutron Flux, P-6	2	1	1	2	3
B. Power Range Neutron Flux, P-8	4	2	3	1	12
C. Power Range Neutron Flux, P-9	4	2	3	1	12
D. Power Range Neutron Flux, P-10	4	2	3	1	12
E. Turbine Impulse Chamber Pressure, P-13	2	1	1	1	12

TABLE 3.3-1 (Continued)

NPF-73

REACTOR TRIP SYSTEM INSTRUMENTATION

FUNCTIONAL UNIT		TOTAL NO. OF CHANNELS	CHANNELS TO TRIP	MINIMUM CHANNELS OPERABLE	APPLICABLE MODES	ACTION
19.	Safety Injection Input from ESF	2	1	2	1, 2	1
20.	Reactor Coolant Pump Breaker Position Trip (Above P-7)	1/breaker	2	1/breaker per operating loop	1	11
21.	Reactor Trip Breakers	2	1	2	1, 2	40
		2	1	2	3 <sup>(3)</sup> , 4 <sup>(3)</sup> , 5 <sup>(3)</sup>	39
22.	Automatic Trip Logic	2	1	2	1, 2	1
		2	1	2	3 <sup>(3)</sup> , 4 <sup>(3)</sup> , 5 <sup>(3)</sup>	39
23.	Reactor Trip System Interlocks					
	a. Intermediate Range Neutron Flux, P-6	2	1	2	2	44
	b. Power Range Neutron Flux, P-8	4	2	3	1	44
	c. Power Range Neutron Flux, P-9	4	2	3	1	44
	d. Power Range Neutron Flux, P-10	4	2	3	1, 2	44
	e. Turbine Impulse Chamber Pressure, P-13	2	1	2	1	44

TABLE 3.3-3 (Continued)

NPF-73

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
4. STEAM LINE ISOLATION					
a. Manual Initiation					
1. Individual	1/steam line	1/steam line	1/operating steam line	1, 2, 3	41
2. System	2 sets (2 switches/set)	1 set	2 sets	1, 2, 3	18
b. Automatic Actuation Logic and Actuation Relays	2	1	2	1, 2, 3	13
c. Containment Pressure Intermediate-High-High	3	2	2	1, 2, 3	14
d. Steamline Pressure-Low	3/loop	2/loop any loop	2/operating loop	1, 2, 3 <sup>(1)</sup>	14
e. Steamline Pressure Rate--High Negative	3/loop	2/loop any loop	2/operating loop	3 <sup>(2)</sup>	14
5. TURBINE TRIP & FEEDWATER ISOLATION					
a. Automatic Actuation Logic and Actuation Relays	2	1	2	1, 2	42
b. Steam Generator Water Level -- High-High, P-14	3/loop	2/loop in any operating loop	2/loop in each operating loop	1, 2, 3	14