10 CFR 50.46(a)(3)(ii)

CP&L

Carolina Power & Light Company
Robinson Nuclear Plant
3581 West Entrance Road
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United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261/LICENSE NO. DPR-23

REPORT OF CHANGES TO OR ERRORS
IN AN ACCEPTABLE LOSS-OF-COOLANT ACCIDENT EVALUATION
MODEL APPLICATION FOR THE EMERGENCY CORE COOLING SYSTEM

Ladies and Gentlemen:

The purpose of this letter is to transmit the Carolina Power & Light (CP&L) Company report of changes to and errors in acceptable Loss-of-Coolant Accident (LOCA) evaluation models (EMs) and EM applications for the Emergency Core Cooling System (ECCS) at the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2. The LOCA EMs are referenced in the Core Operating Limits Report (COLR). Non-significant changes to or errors in EMs and EM applications were previously reported to the NRC by letter dated November 24, 1999.

By letter dated July 20, 2000, CP&L reported to the NRC estimates of significant errors in the application of the Large Break LOCA (LBLOCA) EM. Since CP&L's last annual report of November 24, 1999, there have been no reportable errors or changes in application of the LBLOCA EM other than those reported by CP&L letter dated July 20, 2000.

This report describes a non-significant error in the Siemens Power Corporation (SPC) EXEM Pressurized Water Reactor (PWR) Small Break LOCA (SBLOCA) EM for SBLOCAs. The estimate of the effect of this error in the SBLOCA EM on HBRSEP, Unit No. 2 Peak Clad Temperature (PCT) is summarized in Attachment I.

The latest PCT estimates for the LBLOCA, SBLOCA, and transfer of the ECCS from the Injection Mode to the Recirculation Mode are included in Attachment II.

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The ECCS PCT during the transfer to recirculation has been reanalyzed by SPC and the analysis is currently being reviewed by CP&L. Upon acceptance by CP&L, the results will significantly reduce the reported PCT associated with switchover operation, and the results will be reported to the NRC in accordance with 10 CFR 50.46(a)(3)(ii).

If you have any questions concerning this matter, please contact me or Mr. H. K. Chernoff.

Sincerely,

(acting)

R. L. Warden
Manager - Regulatory Affairs

ALG/alg

Attachments
I. Report of Changes to or Errors in an Acceptable Loss-of-Coolant Accident Evaluation Model for the Emergency Core Cooling System
II. Latest Peak Clad Temperature Estimates

c: L. A. Reyes, USNRC, Region II
R. Subbaratnam, NRC, NRR
NRC Resident Inspector, HBRSEP

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

REPORT OF CHANGES TO OR ERRORS IN AN ACCEPTABLE LOSS-OF-COOLANT ACCIDENT EVALUATION MODEL FOR THE EMERGENCY CORE COOLING SYSTEM

This report provides an estimate of the effect of a non-significant error in the acceptable Loss-of-Coolant Accident (LOCA) evaluation models (EMs) and EM applications for the Emergency Core Cooling System (ECCS) at the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2, since last being reported annually to the NRC by letter dated November 24, 1999.

Non-Significant Error in LOCA EM

Siemens Power Corporation (SPC) identified an error in the flow diversion model relating to fuel rod swelling used for LOCA calculations. The error required modification of the TOODEE2 codes. There was no impact on the Large Break LOCA results. The impact on the Small Break LOCA (SBLOCA) Peak Clad Temperature (PCT) was -1°F. Since the last reported PCT value for the SBLOCA is 2010.6°F, the net effect on PCT, after subtracting the degree, has been rounded up to a new reported value of 2010°F.

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
LATEST PEAK CLAD TEMPERATURE ESTIMATES

The current Peak Clad Temperatures (PCTs) associated with Loss-of-Coolant Accidents (LOCAs) are listed below. The estimates include the cumulative effects of significant and non-significant error corrections and evaluation model changes to date.

The current PCTs associated with Loss-of-Coolant Accident (LOCA) Emergency Core Cooling System (ECCS) Evaluation Models are listed below.

Event	PCT (°F)
Large Break (LB) LOCA ECCS Injection Mode	2143
LBLOCA Transfer to Recirculation Mode	2102

Event	PCT (°F)
Small Break (SB) LOCA ECCS Injection Mode	2010
SBLOCA Transfer to Recirculation Mode	900