

RS-00-139

November 17, 2000

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555-0001

**Braidwood Station, Units 1 and 2**  
**Facility Operating License Nos. NPF-72 and NPF-77**  
**NRC Docket Nos. STN 50-456 and STN 50-457**

**Byron Station, Unit 1**  
**Facility Operating License No. NPF-37**  
**NRC Docket No. STN 50-454**

**Subject: Completion of Modifications to Support the Implementation of Amendment Nos. 108 and 114, Extension of the Allowed Outage Times for the Emergency Diesel Generators, for the Braidwood Station and the Byron Station, Respectively**

**References: (1) Letter from G. F. Dick (USNRC) to O. D. Kingsley (ComEd), "Issuance of Amendments," dated September 1, 2000.**

**(2) Letter from R. M. Krich (ComEd) to USNRC, "Supplement To Request for Amendment to Technical Specifications Extension of Allowable Completion Times and Surveillance Requirement Change for Emergency Diesel Generators," dated April 3, 2000.**

Reference 1 issued Amendment Nos. 108 and 114, for the Braidwood Station and the Byron Station, respectively, related to the Extension of the Allowed Outage Times for the Emergency Diesel Generators. As stated in Reference 1, "Approval of the amendments is based on the information provided to us regarding the current plant configurations as well as

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proposed modifications. The amendments are effective upon the completion of the proposed modifications. We request notification when the modification is complete for each unit." The purpose of this letter is to provide notification of the completion of modifications made in September 2000 at the Braidwood Station, Unit 2 and a shared system, and in August 2000 at the Byron Station, Unit 1.

At Braidwood Station, to eliminate the flooding vulnerability involving the potential for introduction of water into the Auxiliary Building from the Essential Service Water (ESW) System, the ESW System discharge piping was extended such that it terminates above the surface of the cooling lake. To address the consequences to the Reactor Coolant Pump (RCP) seals on a loss of ESW, modifications were made to Braidwood Station, Unit 2 and Byron Station, Unit 1 to provide alternate cooling from the Fire Protection System to the oil coolers and gear boxes on one Centrifugal Charging Pump (CCP) on each unit. The CCP provides seal injection water to the RCP seals. Upon completion of modifications to Braidwood Station, Unit 1 and Byron Station, Unit 2 CCPs, we will provide a similar notification.

Should you have any questions concerning this letter, please contact Ms. Kelly M. Root at 630-663-7292.

Respectfully,



R. M. Krich  
Director, Licensing  
Mid-West Regional Operating Group

cc: Regional Administrator – NRC Region III  
NRC Senior Resident Inspector – Braidwood Station  
NRC Senior Resident Inspector – Byron Station  
Office of Nuclear Facility Safety – Illinois Department Nuclear Safety

**bcc: Braidwood and Byron Station Project Manager, NRC – NRR  
Site Vice President – Braidwood Station  
Site Vice President – Byron Station  
Licensing Vice President  
Nicholas Reynolds – Winston and Strawn  
Regulatory Assurance Manager – Braidwood Station  
Regulatory Assurance Manager – Byron Station  
Manager, Licensing and Compliance – Braidwood and Byron Stations  
Braidwood Nuclear Licensing Administrator  
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