

AmerGen

A PECO Energy/British Energy Company

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November 14, 2000
2130-00-20303

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington DC 20555

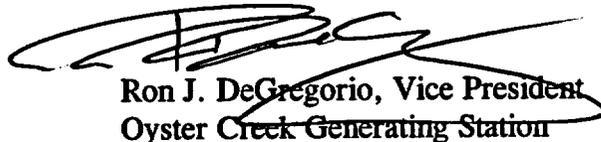
Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Core Operating Limits Report for Cycle 18

Enclosed in this letter is the Core Operating Limits Report for Operating Cycle 18 at the Oyster Creek Generating Station (Topical Report - 066, Revision 11). This information was prepared in accordance with Technical Specification 6.9.1.f.

If any additional information or assistance is required, please contact Mr. John Rogers of my staff at 609.971.4893.

Very truly yours,



Ron J. DeGregorio, Vice President
Oyster Creek Generating Station

RJD/JJR

cc: Administrator, Region I
NRC Senior Project Manager
Senior Resident Inspector

A001

Oyster Creek Cycle 18
Core Operating Limits Report
Topical Report - 066
Rev. 11

BA Number 335400

July 2000

Prepared by: R. V. Furia

Approved: G.R. Bond 7-26-00
Manager Date
Nuclear Fuels

Approved: R.M. Doey 7/27/00
Director Date
Engineering
Support

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This Document Will Not
Be Kept Up To Date
DCC Oyster Creek

TABLE OF CONTENTS

	<u>Page</u>	
Introduction.....	3	
References.....	4	
FIGURE 1 GE9B-348 Fuel MAPLHGR Limits.....	5	
FIGURE 2 GE9B-338 Fuel MAPLHGR Limits.....	6	
FIGURE 3 MCPR Limits.....	7	
FIGURE 4 K_f Flow Factor.....	8	
FIGURE 5 LLHGR Limits.....	9	

INTRODUCTION

Generic Letter 88-16 provides guidance for Technical Specification Changes concerning cycle-specific limits. The generic letter provides a vehicle for the removal of cycle specific parameters from the Technical Specifications and the maintenance of these values within a Core Operating Limits Report (COLR). The Technical Specification modification also establishes reporting requirements and includes definitions supporting the proposed changes. The COLR, including mid-cycle revisions, will be provided for each reload cycle.

This COLR has been prepared in accordance with the requirements of OC Technical Specification 6.9.1.f. The information in this report was reviewed and approved for use at Oyster Creek by means of the Cycle 18 Reload Information and Safety Analysis Report (TR-133, Revision 0) dated July 2000 (Reference 7). The Cycle 18 fuel/core operating limits were generated using the NRC-approved codes and methodologies identified in References 1 through 6.

For each GE fuel design, the APLHGR limits provided in the COLR for operation with less than five loops are calculated to be the same as the five-loop limits at all exposure levels provided a non-operating loop is not an ISOLATED RECIRCULATION LOOP. If a non-operating loop is ISOLATED, both the suction and discharge valves are in the closed position as defined in Reference 6, then a 0.98 MAPHLGR multiplier must be applied at all exposure levels. Only one ISOLATED non-operating loop is permitted. Requirements for operation with recirculation loops out-of-service are provided in Technical Specification 3.3.F.2.

During power operation thermal margins should be maintained within the specified limits. If at any time during power operation it is determined by normal surveillance that the limiting value for APLHGR (Figures 1 and 2), LLHGR (Figure 5) or CPR (Figure 3) is being exceeded, action shall be initiated to restore operation to within the prescribed limits as specified in Technical Specification Section 3.10.

REFERENCES

1. Letter from J. N. Donahew, Jr. (NRC) to P.B. Fiedler (GPUN) dated November 14, 1986, "Reload Topical Report TR 020, Rev 0 (TAC6039)."
2. Letter from A. W. Dromerick (NRC) to P.B. Fiedler (GPUN) dated September 27, 1987, GPU Nuclear Corp. (GPUN) Topical Report TR 021, Revision 0, "Methods for the analysis of Boiling Water Reactors Steady State Physics."
3. Letter from A. W. Dromerick (NRC) to P.B. Fiedler (GPUN) dated March 21, 1988, GPU Nuclear Corp. (GPUN) Topical Report TR 033, Revision 0, "Methods for the Generation of Core Kinetics Data for RETRAN-02 (TAC No. 65138)."
4. Letter from A. W. Dromerick (NRC) to P.B. Fiedler (GPUN) dated March 21, 1988, GPU Nuclear Corp. (GPUN) Topical Report TR 040, Revision 0, "Steady State and Quasi-Steady State Methods for Analyzing Accidents and Transients (TAC No. 65139)."
5. Letter from A. W. Dromerick (NRC) to E.E. Fitzpatrick (GPUN) dated October 12, 1988, GPU Nuclear Corp. (GPUN) Topical Report TR 045, Revision 0, "BWR-2 Transient Analysis Model using the RETRAN Code (TAC No. 66358)."
6. "Oyster Creek NGS SAFER/CORECOOL/GESTR-LOCA Loss-of-Coolant Accident Analysis," NEDE-31462P August 1987
7. "Reload Information and Safety Analysis Report for Oyster Creek Cycle 18 Reload, GPUN TR-133 Revision 0, July 2000
8. Letter from A. W. Dromerick (NRC) to E. E. Fitzpatrick (GPUN) dated October 31, 1988 "Issuance of Amendment No. 129 (TAC No. 67743)."
9. "General Electric Standard Application for Reload Fuel," NEDE-240011-P-A-14 June 2000
10. Letter WHO: 94-036, W.H. Hetzel (GE) to R.V. Furia (GPUN) dated July 29, 1994, "MAPLHGR Report for Oyster Creek Reload Fuel Bundles."

FIGURE 1

GE9B-P8DWB348-12GZ-80M-145-T
MAPLHGR LIMITS

Technical Specification 3.10.A

DATA POINTS

Exposure GWD/MT	LATTICES			
	PSZ KW/FT	DOM KW/FT	SDZ KW/FT	NATU KW/FT
0.22	10.89	10.23	9.81	10.76
1.10	10.97	10.32	9.92	10.59
3.31	11.14	10.61	10.28	10.62
5.51	11.30	10.94	10.68	10.76
8.82	11.26	11.26	11.13	10.93
11.02	11.21	11.21	11.21	10.99
13.78	11.15	11.15	11.15	10.76
16.53	10.83	10.83	10.83	10.38
19.29	10.05	10.05	10.05	9.99
22.05	9.93	9.93	9.93	9.60
27.56	9.74	9.74	9.74	8.81
38.58	9.28	9.45	9.43	7.24
48.12				4.78
49.60	7.85	7.97	7.96	
56.49			5.88	
56.62		5.87		
56.87	5.67			

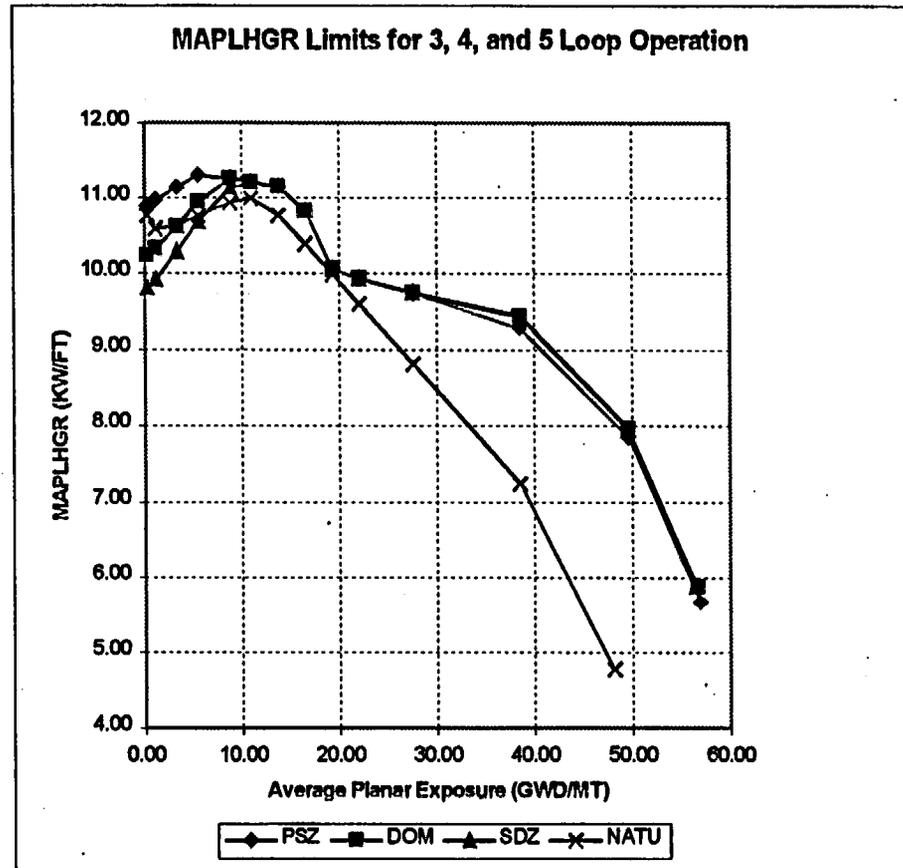


FIGURE 2

GE9B-P8DWB338-11GZ-80M-145-T
MAPLHGR LIMITS

Technical Specification 3.10.A

DATA POINTS

Exposure GWD/MT	LATTICES			
	PSZ KW/FT	DOM KW/FT	SDZ KW/FT	TWZ KW/FT
0.22	10.87	10.30	9.95	10.87
1.10	10.96	10.40	10.05	10.96
3.31	11.19	10.77	10.45	11.19
5.51	11.41	11.23	10.94	11.41
6.61	11.38	11.38	11.18	11.38
7.72			11.34	
8.82	11.31	11.31		11.31
11.02	11.25	11.25	11.25	11.25
16.53	10.65	10.65	10.65	10.65
19.29	10.04	10.04	10.04	10.04
22.05	9.94	9.94	9.94	9.94
27.56	9.77	9.77	9.77	9.77
38.58	9.08	9.27	9.26	9.09
49.60	7.77	7.93	7.92	7.77
56.17				5.80
56.20	5.79			
57.01			5.74	
57.06		5.77		

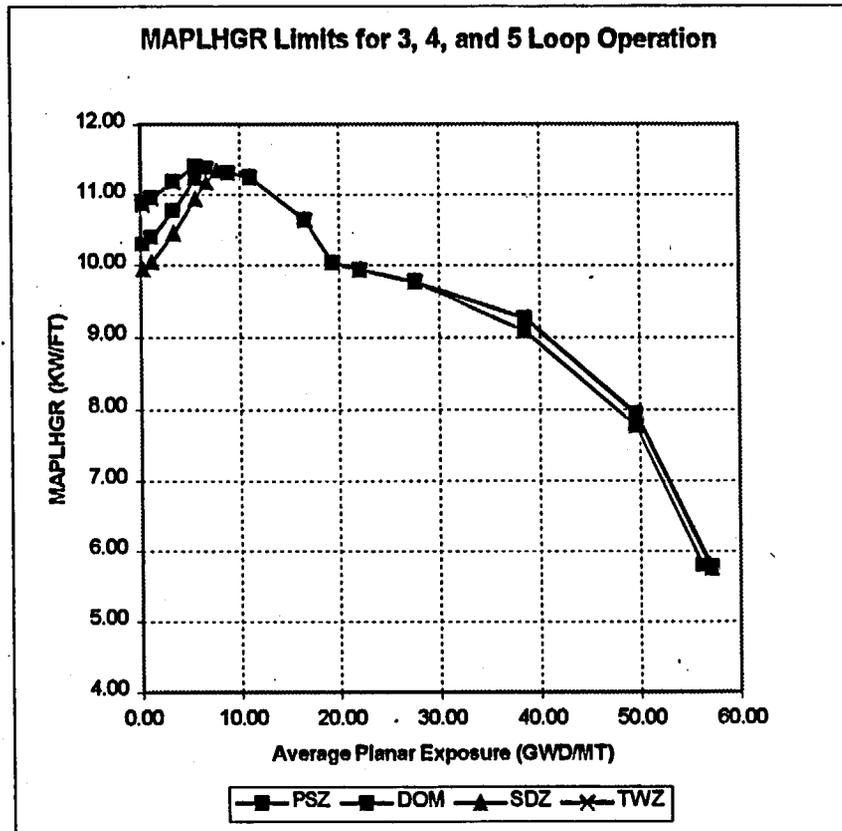


FIGURE 3

MINIMUM CRITICAL POWER RATIO (MCPR) - Tech Spec 3.10.C

APRM STATUS	<u>MCPR LIMIT</u>
1. If any two (2) LPRM assemblies which are input to the APRM system and are separated in distance by less than three (3) times the control rod pitch contain a combination of three (3) out of four (4) detector located in either the A and B or C and D levels which are failed or bypassed (i.e., APRM channel or LPRM input bypassed or inoperable)	1.56
2. If any LPRM input to the APRM system at the B, C, or D level is failed or bypassed or any APRM channel is inoperable (or bypassed).	1.56
3. All B, C, and D LPRM inputs to the APRM system are operating and no APRM channels are inoperable or bypassed.	1.56

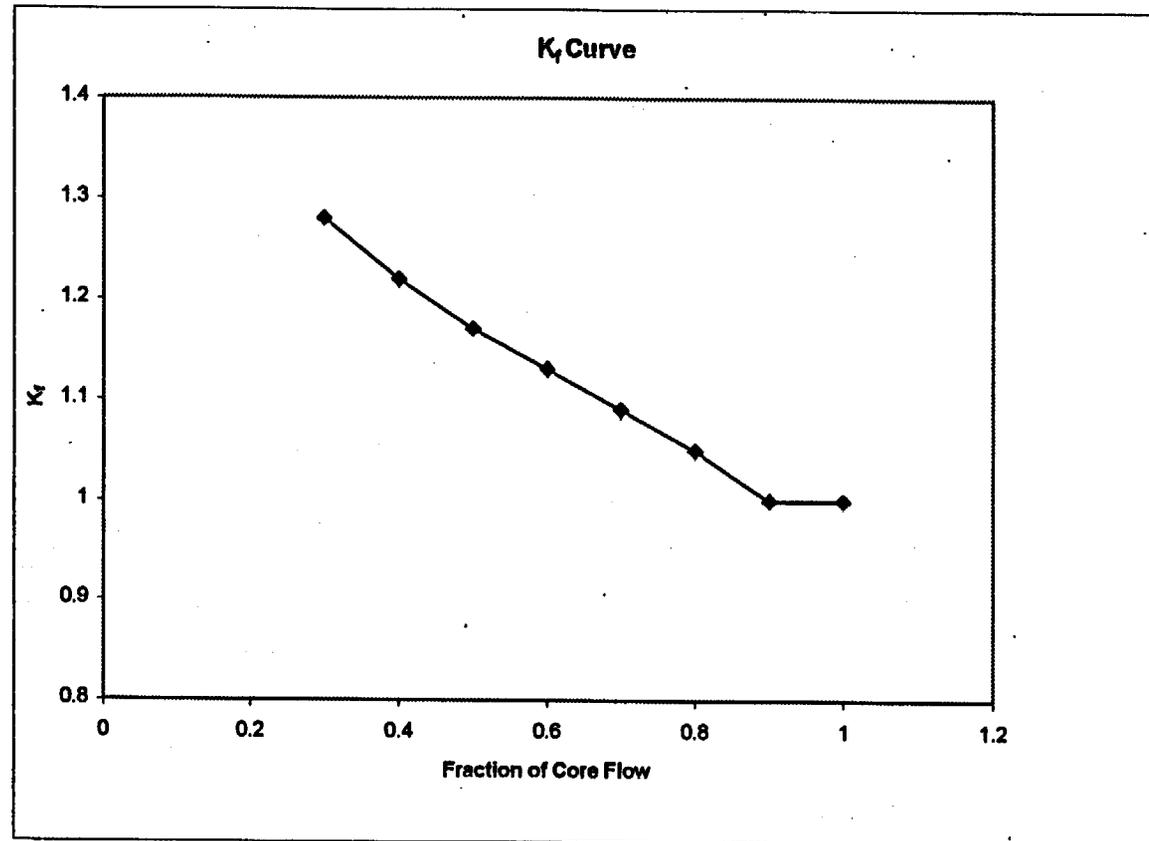
FIGURE 4

Technical Specification 3.10.C

DATA POINTS

FLOW	K_f
0.3	1.28
0.4	1.22
0.5	1.17
0.6	1.13
0.7	1.09
0.8	1.05
0.9	1.00
1.0	1.00

Flowmax = 117.0%



NOTE: For Fraction of Core Flow (FCF) less than 0.40 the following adjustment factor must be applied to the curve: $1.0 + (0.32)(1.22)(0.40 - \text{FCF})$

FIGURE 5

LOCAL LINEAR HEAT GENERATION RATE (LLHGR) - Tech Spec 3.10:B

FUEL TYPE

LLHGR Limit

GE8x8NB

≤ 13.4 kw/ft

Ron J. DeGregorio
Vice President

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Forked River, NJ 08731-0388

2130- 00-20305
November 14, 2000

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Dear Sir:

SUBJECT: Oyster Creek Generating Station
Docket No. 50-219
Monthly Operating Report – October 2000

Enclosed is the October 2000, Monthly Operating Report for the Oyster Creek Generating Station. The content and format of information submitted in this report is in accordance with the guidance provided by Generic Letter 97-02. If you should have any questions, please contact Ms. Brenda DeMerchant, Oyster Creek Regulatory Affairs Engineer, at 609-971-4642.

Very truly yours,



Ron J. DeGregorio
Vice President, Oyster Creek

Enclosures

cc: Administrator, Region I (2 copies)
NRC Project Manager
NRC Sr. Resident Inspector

APPENDIX A
Operating Data Report

Docket No: 50-219
 Date: 11/13/00
 Completed By: Roger B. Gayley
 Telephone: (609)971- 4406

Reporting Period: October 2000

		MONTH	YEAR TO DATE	CUMULATIVE
1.	DESIGN ELECTRICAL RATING (MWe NET). The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.	650	*	*
2.	MAXIMUM DEPENDABLE CAPACITY (MWe NET). The gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions minus the normal station service loads.	619	*	*
3.	NUMBER OF HOURS REACTOR WAS CRITICAL. The total number of hours during the gross hours of the reporting period that the reactor was critical.	305.5	6214.2	192,034.2
4.	HOURS GENERATOR ON LINE. (Service Hours) The total number of hours during the gross hours of the reporting period that the unit operated with the breakers closed to the station bus. The sum of the hours that the generator was on line plus the total outage hours in the reporting period.	303.1	6,008.8	187,946.8
5.	UNIT RESERVE SHUTDOWN HOURS. The total number of hours during the gross hours of the reporting period that the unit was removed from service for economic or similar reasons but was available for operation.	0	0	918.2
6.	NET ELECTRICAL ENERGY (MWH). The gross electrical output of the unit measured at the output terminals of the turbine generator minus the normal station service loads during the gross hours of the reporting period, expressed in megawatt hours. Negative quantities should not be used.	143,797	3,286,536	106,259,127

* Design values have no "Year to Date" or "Cumulative" significance.

Appendix B

Unit Shutdowns

Docket No: 50-219
 Date: 11/13/00
 Completed By: Roger B. Gayley
 Telephone: (609)971- 4406

Reporting Period: October 2000

No.	Date	Type*	Duration (Hours)	Reason ¹	Method of Shutting Down Reactor ²	Cause & Corrective Action to Prevent Recurrence
9	10/13/00	S	440.9	C	2	Scheduled Refueling/Maintenance Outage

*
 F Forced
 S Scheduled

¹
Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensing Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

²
Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

Summary: During October, Oyster Creek generated 143,797 net MWh electric, which was 31.2% of its MDC rating. A refueling/maintenance outage, designated 18R, commenced at 1507 hours, October 13. The outage is nominally scheduled for 32 days.

Ron J. DeGregorio
Vice President

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2130-00-20299

November 14, 2000

Mr. Hubert Miller, Administrator
U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Dear Sir:

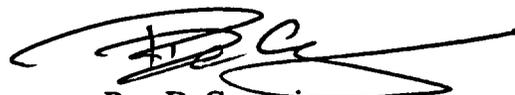
Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Notification of CRO Medical Condition

In accordance with 10 CFR 55.25 we are advising you of the recently diagnosed medical condition of the following CRO which may require an amended license as described in ANSI/ANS-3.4 section 5.3.8.

<u>Name</u>	<u>Docket Number</u>	<u>License Number</u>
Brian Totten	55-60535	OP-10297-3

Following an extended sick leave for mental health issues, Mr. Totten, who remains under medical supervision, has returned to work but has been placed on restricted duty which includes no Licensed Operator duty. We are currently attempting to obtain his medical records from his private physician for your review and determination. It is our intention to forward these records to you as soon as possible. Should you have any questions please contact Brenda DeMerchant, Licensing Engineer at 609.971.4642.

Very Truly Yours,



Ron DeGregorio
Vice President, Oyster Creek

Senior Resident Inspector
Oyster Creek NRC Project Manager