



November 14, 2000

C1100-07  
10 CFR Part 50, Appendix E

Docket No: 50-315

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Mail Stop O-P1-17  
Washington, D. C. 20555-0001

Donald C. Cook Nuclear Plant Unit 1  
REPORT OF CHANGE TO  
EMERGENCY RESPONSE DATA SYSTEM  
DATA POINT CONSTANT

Pursuant to 10 CFR Part 50, Appendix E, Paragraph VI.3.a, Indiana Michigan Power Company (I&M), the licensee for Donald C. Cook Nuclear Plant (CNP), is submitting notification of a change to a data point identified in the Emergency Response Data System (ERDS) Data Point Library. The affected ERDS data point, U1612, indicates total reactor coolant system (RCS) flow. The data point was changed to reflect the increased constant value for RCS flow rate per reactor coolant pump.

Total RCS flow data (data point U1612) is the product of a constant value for RCS flow rate per pump (data point K0200) and a variable data value for normalized RCS flow (data point U0570). During the current outage, the Unit 1 steam generators were replaced with new steam generators of a different design. The replacement steam generator design results in lower tube side flow losses, and consequently, a slightly increased reactor coolant flow. Prior to steam generator replacement, the RCS flow constant was 90,400 gallons per minute (gpm) for each of the four reactor coolant pumps during normal operating conditions. The new RCS flow constant has been calculated to be 94,971 gpm per pump. The plant process computer database has been changed to reflect the increased value for constant point K0200. The revised Data Point Library sheet for the Total RCS Flow data point, U1612, is provided as the attachment to this letter.

No new commitments are made in this letter.

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Should you have any questions concerning this subject, please contact Mr. Wayne J. Kropp, Director of Regulatory Affairs, at (616) 697-5056.

Sincerely,

A handwritten signature in black ink, appearing to read "M. W. Rencheck", with a long horizontal flourish extending to the right.

M. W. Rencheck  
Vice President Nuclear Engineering

/jen

Attachment

c: J. E. Dyer  
J. R. Jolicoeur, NRC - Washington, D.C.  
MDEQ – DW & RPD, w/o attachments  
NRC Resident Inspector  
R. Whale, w/o attachments

ATTACHMENT TO C1100-07

REVISION TO  
DONALD C. COOK NUCLEAR PLANT  
EMERGENCY RESPONSE DATA SYSTEM  
DATA POINT LIBRARY

DATE:	November 14, 2000
REACTOR UNIT:	CK1
DATA FEEDER:	N/A
NRC ERDS PARAMETER:	CORE FLOW
POINT ID:	U1612
PLANT SPEC POINT DESC:	TOTAL RCS FLOW
GENERIC/COND DESC:	TOTAL REACTOR COOLANT FLOW
ANALOG/DIGITAL:	A
ENG UNITS/DIG STATES:	GPM
ENG UNITS CONVERSION:	N/A
MINIMUM INSTR RANGE:	0.0
MAXIMUM INSTR RANGE:	397760.0
ZERO POINT REFERENCE:	N/A
REFERENCE POINT NOTES:	94971 GPM PER LOOP
PROC OR SENS:	P
NUMBER OF SENSORS:	12
HOW PROCESSED:	AVERAGE PER LOOP, SUM OF 4 LOOPS
SENSOR LOCATION:	COLD LEG SUCTION LINE TO RCP'S
ALARM/TRIP SET POINTS:	N/A
NI DETECTOR POWER SUPPLY TURN-OFF POWER LEVEL:	N/A
NI DETECTOR POWER SUPPLY TURN-ON POWER LEVEL:	N/A
INSTRUMENT FAILURE MODE:	N/A
TEMPERATURE COMPENSATION FOR DP TRANSMITTER:	N
LEVEL REFERENCE LEG:	N/A
UNIQUE SYSTEM DESC:	NFP-210, NFP-211, NFP-212, NFP-220, NFP-221, NFP-222, NFP-230, NFP-231, NFP-232, NFP-240, NFP-241, NFP-242