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A PECO Energy/British Energy Company

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November 6, 2000
2130-00-20284

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report 00-010
Local Leak Rate Test Results in Excess of
Technical Specification Limits due to Component Wear

Enclosed is Licensee Event Report LER 00-010. This event did not affect the health and safety of the public.

This LER documents the first excessive leak rate of the Refueling Outage 18R Local Leak Rate Testing Program. A revision to this LER will be submitted when the testing is complete.

If any additional information or assistance is required, please contact Mr. John Rogers of my staff at 609.971.4893.

Very truly yours,

Ron J. DeGregorio
Vice President, Oyster Creek

RJD/JJR
cc: Administrator, Region I
NRC Project Manager
Senior Resident Inspector

IE22

NRC FORM 366 (4-95) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 04/30/98

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Oyster Creek Unit 1	05000 - 219	1 of 3

TITLE (4)

Local Leak Rate Test Results in Excess of Technical Specification Limits due to Component Wear

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER 05000
10	14	2000	2000 --	010 --	00	11	06	2000	FACILITY NAME	DOCKET NUMBER 05000

OPERATING MODE (9)	N
POWER LEVEL (10)	100

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)

	20.2201(b)		20.2203(a)(2)(v)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)
	20.2203(a)(1)		20.2203(a)(3)(i)		50.73(a)(2)(ii)		50.73(a)(2)(x)
	20.2203(a)(2)(i)		20.2203(a)(3)(ii)		50.73(a)(2)(iii)		73.71
	20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		OTHER
	20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		
	20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
John Rogers	609.971.4893

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION	MONTH	DAY	YEAR
X	YES (If yes, complete EXPECTED SUBMISSION DATE).	NO		12	05	2000

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On October 14, 2000, Local Leak Rate Testing (LLRT) results indicated that Main Steam Isolation Valve V-1-0010 exceeded the Technical Specification leak rate limit of .05(.75)La at 35 psig (equivalent to 15.98 SCFH). The leak was quantified as 21.72 SCFH at 35 psig.

The cause of this occurrence was attributed to component wear.

The safety significance of this event is considered minimal as the total penetration leakage would have been limited by Main Steam Isolation Valve V-1-0008 in the same steam header. The leakage past V-1-0008 was quantified at 7.7 SCFH.

V-1-0010 is being refurbished and will be local leak rate tested prior to restart from the current 18R refueling outage.

DATE OF DISCOVERY

The condition being reported was discovered on October 14, 2000.

IDENTIFICATION OF OCCURRENCE

Main Steam Isolation Valve (MSIV) V-1-0010 (EIIS SB-ISV) exceeded the leak rate criteria specified in Technical Specification 4.5.D.2. This condition is considered to be reportable in accordance with 10 CFR 50.73(a)(2)(i).

CONDITIONS PRIOR TO DISCOVERY

The plant was in a cold shutdown condition for a refueling outage when this condition was discovered.

DESCRIPTION OF OCCURRENCE

On October 14, 2000, Local Leak Rate Testing (LLRT) results indicated that Main Steam Isolation Valve V-1-0010 exceeded the Technical Specification leak rate limit of .05(.75) La at 35 psig (equivalent to 15.98 SCFH). The leak was quantified as 21.72 SCFH at 35 psig.

APPARENT CAUSE OF OCCURRENCE

The apparent cause of this occurrence was component wear. This valve is currently being dissassembled and refurbished. The valve will be local leak rate tested prior to restart from the current 18R refueling outage.

This MSIV had previously exceeded Technical Specification limits during testing in the 15R refueling outage, but had successfully passed LLRT in both 16R and 17R.

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		YEAR	SEQUENTIAL NUMBER	REV	
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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

ANALYSIS OF OCCURRENCE AND SAFETY SIGNIFICANCE

The MSIVs are containment isolation valves designed to minimize coolant loss from the vessel, and the resultant offsite dose, in the event of a main steamline break accident. The design basis loss of coolant accident was evaluated at the primary containment maximum allowable accident leak rate of 1.0% per day at an initial pressure of 35 psig which decays to 1.0 psig after 2.5 hours. The 1.0 psig is assumed to remain for the next 21.5 hours. This leakage provides adequate margin between projected potential offsite dose and 10 CFR 100 guidelines. This projected dose was not exceeded.

The analysis for the contribution of MSIV leakage to control room habitability was reviewed. The MSIV contribution is approximately 25% (243 SCF) of the total radioactive leakage assumed (1000SCF). The minor increase in leakage rate from V-1-0010 is well bounded by the existing total radioactive leakage and therefore has no impact on control room habitability.

The safety significance of this event is considered minimal. The leakage past the MSIV would have been limited by the leak rate of the other MSIV in the same header which met the leak rate acceptance criteria of Technical Specification 4.5.D.2.

CORRECTIVE ACTIONS

The MSIV will be inspected and repaired as necessary. A revision to this LER will be submitted to document the final results when the LLRT program is complete.

SIMILAR EVENTS

LER 98-013: Local Leak Rate Test Results in Excess of Technical Specification Limits