



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

WASHINGTON, D.C. 20555-0001

February 9, 1998

Mr. J. E. Cross
President-Generation Group
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

**SUBJECT: BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2
(TAC NOS. M94026, M94027 AND M94953)**

Dear Mr. Cross:

The Commission has issued the enclosed Amendment No. 211 to Facility Operating License No. DPR-66 and Amendment No. 89 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and BVPS-2). These amendments consist of changes to the Technical Specifications (TSs) in response to your applications dated November 6, 1995, and March 11, 1996, as supplemented June 5, 1997, which submitted Proposed Operating License Change Request Nos. 228 (Unit No. 1) and 102 and 103 (Unit No. 2).

These amendments revise the alarm setpoints for the effluent radiation and in-containment area radiation monitors listed in TS Table 3.3-6. These revisions make these alarm setpoints consistent with criteria for the Emergency Action Levels (EALs) approved by the NRC in August 1994. The EALs use these monitors as an indication of fission product barrier challenges or failures. These amendments also revise Action Statement 36 of TS Table 3.3-6 to reflect a previously approved change in reporting frequency (change from semi-annual to annual) for effluent releases. The revision to Action Statement 36 makes it consistent with the previously approved change. These amendments include several editorial changes to the TSs which do not change the intent of the TSs.

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Your application for this amendment addressed the relevant issues; your response to our request for additional information was timely and provided the requested information. The application's no significant hazards consideration determination was suitable for use without changes.

A copy of our Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Donald S. Brinkman, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-334
and 50-412

- Enclosures: 1. Amendment No. 211 to License No. DPR-66
- 2. Amendment No. 89 to License No. NPF-73
- 3. Safety Evaluation

cc w/encls: See next page

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50-334

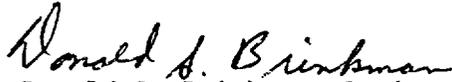
Mr. J. E. Cross

- 2 -

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A copy of our Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,



Donald S. Brinkman, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-334
and 50-412

Enclosures: 1. Amendment No. 211 to License No. DPR-66
2. Amendment No. 89 to License No. NPF-73
3. Safety Evaluation

cc w/encls: See next page

J. E. Cross
Duquesne Light Company

Beaver Valley Power Station
Units 1 & 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 211
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Duquesne Light Company, et al. (the licensee) dated November 6, 1995, as supplemented June 5, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.211, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: February 9, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 211

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 3-34
3/4 3-34a
3/4 3-35
3/4 3-35a

Insert

3/4 3-34
3/4 3-34a
3/4 3-35
3/4 3-35a

TABLE 3.3-6

DPR-66

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>SETPOINT</u> ⁽³⁾	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
1. AREA MONITORS					
a. Fuel Storage Pool Area (RM-207)	1	(1)	≤ 15 mR/hr	10 ⁻¹ - 10 ⁴ mR/hr	19
b. Containment					
i. Purge & Exhaust Isolation (RMVS 104 A & B)	1	6	≤ 1.6 x 10 ³ cpm	10 - 10 ⁶ cpm	22
ii. Area (RM-RM-219 A & B)	2	1,2,3 & 4	≤ 1.5 x 10 ⁴ R/hr	1 - 10 ⁷ R/hr	36
c. Control Room Isolation (RM-RM-218 A & B)	2	1,2,3,4,5 ⁽⁴⁾ ,6 ⁽⁴⁾ (in either unit)	≤ .47 mR/hr	10 ⁻² - 10 ³ mR/hr	41
2. PROCESS MONITORS					
a. Containment					
i. Gaseous Activity RCS Leakage Detection (RM 215B)	1	1,2,3 & 4	N/A	10 - 10 ⁶ cpm	20
ii. Particulate Activity RCS Leakage Detection (RM 215A)	1	1,2,3 & 4	N/A	10 - 10 ⁶ cpm	20
b. Fuel Storage Building Gross Activity (RMVS-103 A & B)	1	(2)	≤ 4.0 x 10 ⁴ cpm	10 - 10 ⁶ cpm	21

TABLE 3.3-6 (Continued)

DPR-66

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>SETPOINT</u> ⁽³⁾	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
PROCESS MONITORS (Continued)					
2.c. Noble Gas Effluent Monitors					
i. Supplementary Leak Collection and Release System (RM-VS-110 Ch. 7 & Ch. 9) ⁽⁷⁾	1	1,2,3,&4	$\leq 7.98 \times 10^2 \text{cpm}$	$10^{-2}-10^5 \text{uCi/cc}^{(5)}$	36
ii. Auxiliary Building Ventilation System (RM-VS-109 Ch. 7 & Ch. 9) ⁽⁷⁾	1	1,2,3,&4	$\leq 6.69 \times 10^2 \text{cpm}$	$10^{-2}-10^5 \text{uCi/cc}^{(5)}$	36
iii. Process Vent System (RM-GW-109 Ch. 7 & Ch. 9) ⁽⁷⁾	1	1,2,3,&4	$\leq 1.83 \times 10^5 \text{cpm}$	$10^{-2}-10^5 \text{uCi/cc}^{(6)}$	36
iv. Atmospheric Steam Dump Valve and Code Safety Relief Valve Discharge (RM-MS-100 A, B, C)	1/SG	1,2,3,&4	$\leq 5.0 \times 10^1 \text{cpm}$	$10^{-1}-10^3 \text{uCi/cc}$	36
v. Auxiliary Feedwater Pump Turbine Exhaust (RM-MS-101)	1	1,2,3,&4	$\leq 6.5 \times 10^2 \text{cpm}$	$10^{-1}-10^3 \text{uCi/cc}$	36

TABLE NOTATIONS

- (1) With fuel in the storage pool or building.
- (2) With Irradiated fuel in the storage pool.
- (3) Above background.
- (4) During movement of irradiated fuel or movement of heavy loads over spent fuel.
- (5) Nominal range for Ch. 7 and Ch. 9. Alarm set on Ch. 7.
- (6) Nominal range for Ch. 7 and Ch. 9. Alarm set on Ch. 9.
- (7) Other SPING-4 channels not applicable to this specification.

ACTION STATEMENTS

- ACTION 19 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, perform area surveys of the monitored area with portable monitoring instrumentation at least once per 24 hours.
- ACTION 20 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.4.6.1.
- ACTION 21 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the applicable ACTION requirements of Specification 3.9.12 and 3.9.13.
- ACTION 22 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.9.
- ACTION 36 - With the number of OPERABLE channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable Channel(s) to OPERABLE status within 72 hours, or:
- a) Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and
 - b) Return the channel to OPERABLE status within 30 days, or, explain in the next Annual Radioactive Effluent Release Report why the inoperability was not corrected in a timely manner.
- ACTION 41 - a) With the number of Unit 1 OPERABLE channels one less than the Minimum Channels OPERABLE requirement:
- 1. Verify the respective Unit 2 control room radiation monitor train is OPERABLE within 1 hour and at least once per 31 days.

TABLE 3.3-6 (Continued)ACTION STATEMENTS

ACTION 41 (Continued)

2. With the respective Unit 2 control room radiation monitor train inoperable, suspend all operations involving movement of irradiated fuel within 1 hour and restore the Unit 1 control room radiation monitor to OPERABLE status within 7 days or isolate the control room from the outside atmosphere by closing all series air intake and exhaust isolation dampers, unless the respective Unit 2 control room radiation monitor train is restored to OPERABLE status within 7 days.
- b) With no Unit 1 control room radiation monitors OPERABLE:
1. Verify both Unit 2 control room radiation monitors are OPERABLE within 1 hour and at least once per 31 days.
 2. With either Unit 2 control room radiation monitor inoperable, suspend all operations involving movement of irradiated fuel within 1 hour and restore the respective Unit 1 control room radiation monitor train to OPERABLE status within 7 days or isolate the control room from the outside atmosphere by closing all series air intake and exhaust isolation dampers, unless the respective Unit 2 control room radiation monitor train is restored to OPERABLE status within 7 days.
 3. With no Unit 2 control room radiation monitors OPERABLE, immediately isolate the combined control room by closing all series air intake and exhaust isolation dampers and be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 89
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Duquesne Light Company, et al. (the licensee) dated November 6, 1995, and March 11, 1996, as supplemented June 5, 1997, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

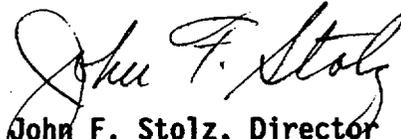
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 89 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DLCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: February 9, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 89

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 3-40
3/4 3-41
3/4 3-42

Insert

3/4 3-40
3/4 3-41
3/4 3-42
3/4 3-42a

TABLE 3.3-6

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>SETPOINT</u> ⁽³⁾	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
1. AREA MONITORS					
a. Fuel Storage Pool Area (2RMF-RQ202)	1	(1)	≤75.8 mR/hr	10 ⁻¹ to 10 ⁴ mR/hr	19
b. Containment Area (2RMR-RQ206 & 207)	2	1, 2, 3 & 4	≤2.0x10 ⁴ R/hr	1 to 10 ⁷ R/hr	36
c. Control Room Area (2RMC-RQ201 & 202)	2	1, 2, 3 & 4, 5 ⁽⁴⁾ & 6 ⁽⁴⁾	≤0.476 mR/hr	10 ⁻² to 10 ³ mR/hr	46, 47
2. PROCESS MONITORS					
a. Containment					
i. Gaseous Activity (Xe-133) RCS Leakage Detection (2RMR-RQ303B)	1	1, 2, 3 & 4	N/A	10 ⁻⁶ to 10 ⁻¹ μCi/cc	20
ii. Particulate Activity (I-131) RCS Leakage Detection (2RMR-RQ303A)	1	1, 2, 3 & 4	N/A	10 ⁻¹⁰ to 10 ⁻⁵ μCi/cc	20
b. Fuel Building Vent					
i. Gaseous Activity (Xe-133) (2RMF-RQ301B)	1	(2)	≤7.82x10 ⁻⁶ μCi/cc	10 ⁻⁶ to 10 ⁻¹ μCi/cc	21

TABLE 3.3-6 (Continued)

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>SETPOINT</u> ⁽³⁾	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
2. PROCESS MONITORS (Continued)					
ii. Particulate (I-131) (2RMF-RQ301A)	1	(2)	$\leq 6.70 \times 10^{-9} \mu\text{Ci/cc}$	10^{-10} to $10^{-5} \mu\text{Ci/cc}$	21
c. Noble Gas and Effluent Monitors					
i. Supplementary Leak Collection and Release System					
1) Mid Range Noble Gas (Xe-133) (2HVS-RQ109C)	1	1,2,3&4	N.A.	10^{-4} to $10^2 \mu\text{Ci/cc}$	36
2) High Range Noble Gas (Xe-133) (2HVS-RQ109D)	1	1,2,3&4	N.A.	10^{-1} to $10^5 \mu\text{Ci/cc}$	36
ii. Containment Purge Exhaust (Xe-133) (2HVR-RQ104A & B)	1	6	$\leq 1.01 \times 10^{-3} \mu\text{Ci/cc}$	10^{-6} to $10^{-1} \mu\text{Ci/cc}$	22
iii. Main Steam Discharge (Kr-88) (2MSS-RQ101A, B & C)	1/SG	1,2,3&4	$\leq 3.9 \times 10^{-2} \mu\text{Ci/cc}$	10^{-2} to $10^3 \mu\text{Ci/cc}$	36

TABLE 3.3-6 (Continued)TABLE NOTATIONS

- (1) With fuel in the storage pool or building.
- (2) With irradiated fuel in the storage pool.
- (3) Above background.
- (4) During movement of irradiated fuel.

ACTION STATEMENTS

- ACTION 19 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, perform area surveys of the monitored area with portable monitoring instrumentation at least once per 24 hours.
- ACTION 20 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.4.6.1.
- ACTION 21 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the applicable ACTION requirements of Specifications 3.9.12 and 3.9.13.
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- ACTION 36 - With the number of OPERABLE channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable channel(s) to OPERABLE status within 72 hours, or:
- 1) Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and
 - 2) Return the channel to OPERABLE status within 30 days, or, explain in the next Annual Radioactive Effluent Release Report why the inoperability was not corrected in a timely manner.

TABLE 3.3-6 (Continued)ACTION STATEMENTS (Continued)

- ACTION 46 - With the number of OPERABLE channels one less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable channel to OPERABLE status within 7 days or close the control room series normal air intake and exhaust isolation dampers.
- ACTION 47 - With no OPERABLE channels either restore one inoperable channel to OPERABLE status within 1 hour or close the control room series normal air intake and exhaust isolation dampers.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 211 AND 89 TO FACILITY OPERATING
LICENSE NOS. DPR-66 AND NPF-73
DUQUESNE LIGHT COMPANY
OHIO EDISON COMPANY
PENNSYLVANIA POWER COMPANY
THE CLEVELAND ELECTRIC ILLUMINATING COMPANY
THE TOLEDO EDISON COMPANY
BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-334 AND 50-412

1.0 INTRODUCTION

By letters dated November 6, 1995, and March 11, 1996, as supplemented June 5, 1997, the Duquesne Light Company (the licensee) submitted a request for changes to the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and BVPS-2), Technical Specifications (TSs). The requested changes would change:

1. The reporting period of Action 36 of TS Table 3.3-6 of BVPS-1 and BVPS-2 from semi-annually to annually, for consistency with the current reporting interval for effluents,
2. The high alarm high setpoints of the BVPS-1 in-containment high range area radiation monitors (RM-RM-219 A & B), from 30 R/hr to 1.5×10^4 R/hr,
3. The high alarm high setpoints for the BVPS-2 in-containment high range area radiation monitors (2RM-RQ-206 and 207) from 3,290 to 20,000 R/hr,
4. The alarm setpoints of the BVPS-1 noble gas effluent monitors (RM-VS-109 from 275 to 669 cpm; RM-VS-110 from 350 to 798 cpm; and RM-GW-109 from 18,000 to 183,000 cpm), and
5. The alarm setpoints of the BVPS-2 containment purge exhaust monitors (2HVR-RQ 104 A & B) from 3 times background to 0.00101 $\mu\text{Ci}/\text{cc}$.
6. Make minor editorial changes on TS Table 3.3-6 (both units).

The June 5, 1997, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination or expand the amendment request beyond the scope of the December 20, 1995, and April 10, 1996, Federal Register notices.

2.0 EVALUATION

Item 1, the change in reporting frequency of Action 36 of TS Table 3.3-6 (both units) from semi-annually to annually would make this reporting frequency consistent with the reporting requirements previously approved in License Amendment Nos. 188 (BVPS-1) and 70 (BVPS-2) but inadvertently not made in Action 36 during the processing of those amendments. The proposed reporting frequency (annually) is consistent with current NRC requirements. Therefore, this change is acceptable.

Items 2 and 3, the changes in the high alarm high setpoints for the in-containment area radiation monitors, are intended to make the high alarm high setpoints correspond to the Emergency Action Level (EAL) for a General Emergency based on radioactivity released into containment. The current BVPS-1 and BVPS-2 EALs were approved by the NRC in August 1994. The NRC staff review confirms that the proposed setpoints are appropriate for the recommended 20-percent fuel damage which corresponds to a General Emergency EAL. The NRC staff notes that although these monitors are safety-related, they do not actuate any safety function nor interface with any other safety system. Therefore, the setpoint changes will not affect actual releases if an accident were to occur, nor will the changes impact releases during normal operations or occupational radiation exposure. These changes, therefore, are acceptable.

Item 4, the changes in the high alarm setpoints for three noble gas effluent monitors (actually three noble gas monitor components of the SPING system) are intended to make the alarm setpoints consistent with the current EALs. These high-range effluent monitors were required as a result of the TMI Lessons Learned effort [2] but no alarm setpoint was stipulated. The setpoint was established to call attention to releases to the environment that correspond to a General Emergency. No safety systems are actuated by these alarms. In 1992, the NRC provided new guidance for EALs [1]. A principal change was the use of annual average meteorological dispersion for the dose calculations. In August 1994, the BVPS-1 and BVPS-2 EALs were revised to meet the NRC guidance and were approved by the NRC. Because the revised EALs have been approved and the proposed changes are consistent with the new EALs, the proposed changes are acceptable.

Item 5, the change in the alarm setpoints of the BVPS-2 containment purge exhaust monitors, is intended to raise the setpoints to a level which will avoid inadvertent alarms and, yet, will be low enough to enable the system to serve its intended purposes. The principal purpose of these monitors is to actuate containment isolation and, thereby, to minimize the offsite and control room doses that could result from a fuel handling accident inside containment. The licensee reports, and the NRC staff has verified, that the

proposed setpoints are low enough to ensure that the relevant criteria are met, i.e. the offsite doses would be kept well below the criteria of 10 CFR Part 100 and the doses in the control room would not exceed the limits of General Design Criterion 19. The secondary objective is to ensure, with the system operational and containment isolated, instantaneous dose rates off site are below the 10 CFR Part 50, Appendix I criteria of 500 mrem/yr to the whole body and 3,000 mrem to the skin. The licensee showed, and the NRC staff verified, that the proposed setpoints are sufficiently low to meet this objective. The NRC staff concludes that the proposed setpoints are low enough to meet the accident mitigation criteria while being high enough to substantially reduce the frequency of inadvertent actuation of an engineered safety feature. These proposed changes to the technical specifications are acceptable.

The proposed editorial changes do not change the intent or the requirements of any of the TSs and are, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (60 FR 65677 and 61 FR 15988). The amendments also relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Willis

Date: February 9, 1998

6.0 REFERENCES

1. **Emergency Planning and Preparedness for Nuclear Power Reactors, U.S. Nuclear Regulatory Commission, Regulatory Guide 1.101, Revision 3, August 1992**
2. **Clarification of TMI Action Plan Requirements, Nuclear Regulatory Commission Report NUREG-0737, November 1980**