1997

April 14, 199

Mr. J. E. Cross
President-Generation Group
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2 (TAC NOS. M96559 AND M96560)

Dear Mr. Cross:

The Commission has issued the enclosed Amendment No. 202 to Facility Operating License No. DPR-66 and Amendment No. 83 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated September 9, 1996, which submitted Proposed Operating License Change Request Nos. 234 and 107.

These amendments modify the design features section (Section 5.0) of the TSs to make the design features section consistent with the intent of 10 CFR 50.36 and with the guidance provided in the NRC's Standard Technical Specifications, Westinghouse Plants (NUREG-1431, Revision 1).

A copy of our Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,
/S/
Donald S. Brinkman, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-334/412

Enclosures:
1. Amendment No. 202 to License No. DPR-66
2. Amendment No. 83 to License No. NPF-73
3. Safety Evaluation

cc w/encls: See next page

NRC FILE CENTER COPY

DF01

DISTRIBUTION:

Docket File	MO'Brien	CGrimes
PUBLIC	DBrinkman	JStolz
PDI-2 Reading	OGC	ACRS
SVarga	GHill(4)	PEselgroth, RGN-I

230047

*Previously Concurred

OFFICE	PDI-2/LA	PDI-2/PM	OGC*	PDI-2/D
NAME	MO'Brien	DBrinkman:cdw	CMarco	JStolz
DATE	4/11/97	4/11/97	04/03/97	4/14/97

OFFICIAL RECORD COPY
DOCUMENT NAME: S:\BV96559.AMD

9704230239 970414
PDR ADOCK 05000334
P PDR

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 14, 1997

Mr. J. E. Cross
President-Generation Group
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2 (TAC NOS. M96559 AND M96560)

Dear Mr. Cross:

The Commission has issued the enclosed Amendment No. 202 to Facility Operating License No. DPR-66 and Amendment No. 83 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated September 9, 1996, which submitted Proposed Operating License Change Request Nos. 234 and 107.

These amendments modify the design features section (Section 5.0) of the TSs to make the design features section consistent with the intent of 10 CFR 50.36 and with the guidance provided in the NRC's Standard Technical Specifications, Westinghouse Plants (NUREG-1431, Revision 1).

A copy of our Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Donald S. Brinkman

Donald S. Brinkman, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-334/412

Enclosures:
1. Amendment No. 202 to License No. DPR-66
2. Amendment No. 83 to License No. NPF-73
3. Safety Evaluation

cc w/encls: See next page

J. E. Cross
Duquesne Light Company

Beaver Valley Power Station
Units 1 & 2

cc:

Jay E. Silberg, Esquire
Shaw, Pittman, Potts & Trowbridge
2300 N Street, NW.
Washington, DC 20037

R. K. Brosi, Manager
Nuclear Safety Department (BV-A)
Duquesne Light Company
Beaver Valley Power Station
PO Box 4
Shippingport, PA 15077

Commissioner Roy M. Smith
West Virginia Department of Labor
Building 3, Room 319
Capitol Complex
Charleston, WVA 25305

John D. Borrows
Director, Utilities Department
Public Utilities Commission
180 East Broad Street
Columbus, OH 43266-0573

Director, Pennsylvania Emergency
Management Agency
Post Office Box 3321
Harrisburg, PA 17105-3321

Ohio EPA-DERR
ATTN: Zack A. Clayton
Post Office Box 1049
Columbus, OH 43266-0149

Dr. Judith Johnsrud
National Energy Committee
Sierra Club
433 Orlando Avenue
State College, PA 16803

Bureau of Radiation Protection
Pennsylvania Department of
Environmental Resources
ATTN: R. Barkanic
Post Office Box 2063
Harrisburg, PA 17120

Mayor of the Borrough of
Shippingport
Post Office Box 3
Shippingport, PA 15077

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 298
Shippingport, PA 15077

Duquesne Light Company
Beaver Valley Power Station
PO Box 4
Shippingport, PA 15077
ATTN: S. C. Jain, Vice President
Nuclear Services (BV-A)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 202
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated September 9, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9704230244 970414
PDR ADOCK 05000334
P PDR

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 202 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

   In addition, the license is amended to add paragraph 2.C.(10) to the Facility Operating License No. DPR-66 as follows:

   (10) Additional Conditions

   The Additional Conditions contained in Appendix C, as revised through Amendment No. 202, are hereby incorporated into this license. Duquesne Light Company shall operate the facility in accordance with the Additional Conditions.

3. This license amendment is effective as of its date of issuance and the change to the facility shall be implemented within 60 days from the date of issuance. Implementation of this amendment shall include the relocation of certain technical specification requirements to the appropriate licensee-controlled documents as described in the Licensee's application dated September 9, 1996, and evaluated in the staff's safety evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION

Chester Poslusny for

John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments: 1. Page 6a to License* No. DPR-66 and page 1 to Appendix C of License* No. DPR-66
2. Changes to the Technical Specifications

Date of Issuance: April 14, 1997

* Page 6a of the license and page 1 of Appendix C are attached, for convenience, for the composite license to reflect this change.

ATTACHMENT TO LICENSE AMENDMENT NO. 202

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

1. Revise License as follows:

Remove Page	Insert Page
6a	6a

2. Insert page 1 of Appendix C of the License

3. Replace the following pages of Appendix A Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	Insert
XIV	XIV
XV	XV
XVI	XVI
XVII	-
XVIII	XVII
XIX	XVIII
XX	XIX
XXI	XX
5-1	5-1
5-1a	-
5-1b	-
5-1d	-
5-1e	-
5-2	-
5-3	-
5-4	-
5-5	-
5-6	5-2

(10) Additional Conditions

The Additional Conditions contained in Appendix C, as revised through Amendment No. 202, are hereby incorporated into this license. Duquesne Light Company shall operate the facility in accordance with the Additional Conditions.

2.D. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the following Commission approved documents, including amendments and changes made pursuant to the authority of 10 CFR 50.54(p). These approved documents consist of information withheld from public disclosure pursuant to 10 CFR 2.790(d):

(1) "Beaver Valley Power Station, Unit No. 1, Physical Security Plan", Revision 1, dated October 31, 1977, as supplemented by Revision 2 dated May 15, 1978, Revision 3 dated February 21, 1979.

(2) The NRC approved Safeguards Contingency Plan as transmitted by letter of March 22, 1980.

(3) "Beaver Valley Power Station Guard Training and Qualification Plan", submitted by letter, dated August 16, 1979, as revised by letter dated September 26, 1980. This Plan shall be fully implemented in accordance with 10 CFR 73.55(b)(4), within 60 days of this approval by the Commission. All security personnel shall be qualified within two years of this approval.

E. All work and activities in connection with this project shall be performed pursuant to the provisions of the Commonwealth of Pennsylvania Clean Streams Acts of June 24, 1913, as amended, and of June 22, 1937, as amended, and in accordance with all permits issued by the Department of Environmental Resources of the Commonwealth of Pennsylvania.

2.F. This amended license is effective as of the date of issuance and shall expire at midnight on January 29, 2016.

FOR THE NUCLEAR REGULATORY COMMISSION

ORIGINAL SIGNED BY
R. S. BOYD

Roger S. Boyd, Director
Division of Project Management
Office of Nuclear Reactor Regulation

Date of Issuance: July 2, 1976

APPENDIX C

ADDITIONAL CONDITIONS
OPERATING LICENSE NO. DPR-66

Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company shall comply with the following conditions on the schedules noted below:

Amendment Number	Additional Condition	Implementation Date
202	The licensee is authorized to relocate certain Technical Specification requirements to licensee-controlled documents. Implementation of this amendment shall include the relocation of these technical specification requirements to the appropriate documents, as described in the licensee's application dated September 9, 1996, and evaluated in the staff's safety evaluation attached to this amendment.	The amendment shall be implemented within 60 days from April 14, 1997

INDEX

BASES

SECTION		PAGE
3/4.11.2	GASEOUS EFFLUENTS	
3/4.11.2.5	Gas Storage Tanks	B 3/4 11-1
3/4.11.2.6	Explosive Gas Mixture	B 3/4 11-1

DESIGN FEATURES

SECTION		PAGE
5.1 SITE LOCATION		5-1
5.2 REACTOR CORE		5-1
5.3 FUEL STORAGE		5-1

ADMINISTRATIVE CONTROLS

SECTION		PAGE
6.1 RESPONSIBILITY		6-1
6.2 ORGANIZATION		
6.2.1	Onsite and Offsite Organizations	6-1
6.2.2	Unit Staff	6-2
6.3 FACILITY STAFF QUALIFICATIONS		6-5
6.4 TRAINING		6-5
6.5 REVIEW AND AUDIT		
6.5.1	ONSITE SAFETY COMMITTEE (OSC)	
6.5.1.1	Function	6-5
6.5.1.2	Composition	6-5
6.5.1.3	Alternates	6-6

INDEX

ADMINISTRATIVE CONTROLS

SECTION		PAGE
6.5.1.4	Meeting Frequency	6-6
6.5.1.5	Quorum	6-6
6.5.1.6	Responsibilities	6-6
6.5.1.7	Authority	6-7
6.5.1.8	Records	6-7
6.5.2	OFFSITE REVIEW COMMITTEE (ORC)	
6.5.2.1	Function	6-7
6.5.2.2	Composition	6-8
6.5.2.3	Alternates	6-8
6.5.2.4	Consultants	6-8
6.5.2.5	Meeting Frequency	6-9
6.5.2.6	Quorum	6-9
6.5.2.7	Review	6-9
6.5.2.8	Audits	6-10
6.5.2.9	Authority	6-11
6.5.2.10	Records	6-11
6.6 REPORTABLE EVENT ACTION		6-11
6.7 SAFETY LIMIT VIOLATION		6-11
6.8 PROCEDURES		6-12
6.9 REPORTING REQUIREMENTS		6-16
6.9.1	Routine Reports	6-16
6.9.1.1,2,3	Startup Reports	6-16

INDEX

ADMINISTRATIVE CONTROLS

SECTION		PAGE
6.9.1.4,5	Annual Reports	6-17
6.9.1.6	Monthly Operating Report	6-18
6.9.1.10	Annual Radiological Environmental Operating Report	6-18
6.9.1.11	Annual Radioactive Effluent Release Report	6-19
6.9.1.12	Core Operating Limits Report	6-19
6.9.2	SPECIAL REPORTS	6-20
6.10	RECORD RETENTION	6-21
6.11	RADIATION PROTECTION PROGRAM	6-22
6.12	HIGH RADIATION AREA	6-23
6.13	PROCESS CONTROL PROGRAM (PCP)	6-24
6.14	OFFSITE DOSE CALCULATION MANUAL (ODCM)	6-24
6.16	MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS	6-25
6.17	CONTAINMENT LEAKAGE RATE TESTING PROGRAM	6-25

TABLE INDEX

TABLE	TITLE	PAGE
2.2-1	Reactor Trip System Instrumentation Trip Setpoints	2-6
3.1-1	Accident Analyses Requiring Reevaluation in the event of an Inoperable Full or Part Length Rod	3/4 1-19a
3.2-1	DNB Parameters	3/4 2-13
3.3-1	Reactor Trip System Instrumentation	3/4 3-2
3.3-2	Reactor Trip System Instrumentation Response Times	3/4 3-9
4.3-1	Reactor Trip System Instrumentation Surveillance Requirements	3/4 3-11
3.3-3	Engineered Safety Features Actuation System Instrumentation	3/4 3-15
3.3-4	Engineered Safety Features Actuation System Instrumentation Trip Setpoints	3/4 3-22
3.3-5	Engineered Safety Feature Response Times	3/4 3-25
4.3-2	Engineered Safety Feature Actuation System Instrumentation Surveillance Requirements	3/4 3-29
3.3-6	Radiation Monitoring Instrumentation	3/4 3-34
4.3-3	Radiation Monitoring Instrumentation Surveillance Requirements	3/4 3-36
3.3-7	Seismic Monitoring Instrumentation	3/4 3-39
4.3-4	Seismic Monitoring Instrumentation Surveillance Requirements	3/4 3-40
3.3-8	Meteorological Monitoring Instrumentation	3/4 3-42
4.3-5	Meteorological Monitoring Instrumentation Surveillance Requirements	3/4 3-43
3.3-9	Remote Shutdown Panel Monitoring Instrumentation	3/4 3-45
4.3-6	Remote Shutdown Monitoring Instrumentation Surveillance Requirements	3/4 3-46

TABLE	TITLE	PAGE
3.3-11	Accident Monitoring Instrumentation	3/4 3-51
4.3-7	Accident Monitoring Instrumentation Surveillance Requirements	3/4 3-52
3.3-13	Explosive Gas Monitoring Instrumentation	3/4 3-55
4.3-13	Explosive Gas Monitoring Instrumentation Surveillance Requirements	3/4 3-57
4.4-1	Minimum Number of Steam Generators to be Inspected During Inservice Inspection	3/4 4-10e
4.4-2	Steam Generator Tube Inspection	3/4 4-10f
4.4-3	Reactor Coolant System Pressure Isolation Valves	3/4 4-14b
3.4-1	Reactor Coolant System Chemistry Limits	3/4 4-16
4.4-10	Reactor Coolant System Chemistry Limits Surveillance Requirements	3/4 4-17
4.4-12	Primary Coolant Specific Activity Sample and Analysis Program	3/4 4-20
3.7-1	Maximum Allowable Power Range Neutron Flux High Setpoint With Inoperable Steamline Safety Valves During 3 Loop Operation	3/4 7-2
3.7-2	Maximum Allowable Power Range Neutron Flux High Setpoint with Inoperable Steam Line Safety Valves During 2 Loop Operation	3/4 7-3
3.7-3	Steam Line Safety Valves Per Loop	3/4 7-4
4.7-1	Snubber Visual Inspection Interval	3/4 7-31
4.7-2	Secondary Coolant System Specific Activity Sample and Analysis Program	3/4 7-9
3.8-1	Battery Surveillance Requirements	3/4 8-9a
3.9-1	Beaver Valley Fuel Assembly Minimum Burnup vs. Initial U235 Enrichment For Storage in Region 2 Spent Fuel Racks	3/4 9-15

TABLE	TITLE	PAGE
B 3/4.4-1	Reactor Vessel Toughness Data (unirradiated)	B 3/4 4-7
6.2-1	Minimum Shift Crew Composition	6-4

Figure Index

FIGURE	TITLE	PAGE
2.1-1	Reactor Core Safety Limit - Three Loop Operation	2-2
3.4-1	Dose Equivalent I-131 Primary Coolant Specific Activity Limit Versus Percent of Rated Thermal Power with the Primary Coolant Specific Activity > 1.0 μCi/gram Dose Equivalent I-131	3/4 4-21
3.4-2	Beaver Valley Unit 1 Reactor Coolant System Heatup Limitations Applicable for the First 16.0 EFPY	3/4 4-24
3.4-3	Beaver Valley Unit 1 Reactor Coolant System Cooldown Limitations Applicable for the First 16.0 EFPY	3/4 4-25
3.6-1	Maximum Allowable Primary Containment Air Pressure Versus River Water Temperature	3/4 6-7
B 3/4.2-1	Typical Indicated Axial Flux Difference Versus Thermal Power at BOL	B 3/4 2-3
B 3/4.4-1	Fast Neutron Fluence (E>1 Mev) as a Function of Full Power Service Life	B 3/4 4-6a
B 3/4.4-2	Effect of Fluence, Copper Content, and Phosphorus Content on ΔRT_{NDT} for Reactor Vessel Steels per Reg. Guide 1.99	B 3/4 4-6b
B 3/4.4-3	Isolated Loop Pressure-Temperature Limit Curve	B 3/4 4-10a

5.1 SITE LOCATION

The Beaver Valley Power Station Unit No. 1 is located in Shippingport Borough, Beaver County, Pennsylvania, on the south bank of the Ohio River. The site is approximately 1 mile southeast of Midland, Pennsylvania, 5 miles east of East Liverpool, Ohio, and approximately 25 miles northwest of Pittsburgh, Pennsylvania. The exclusion area boundary has a minimum radius of 2000 feet from the center of containment.

5.2 REACTOR CORE

5.2.1 FUEL ASSEMBLIES

The reactor shall contain 157 fuel assemblies. Each assembly shall consist of a matrix of zircaloy clad fuel rods with an initial composition of natural or slightly enriched uranium dioxide (UO_2) as fuel material. Limited substitutions of zirconium alloy or stainless steel filler rods for fuel rods, in accordance with approved applications of fuel rod configurations, may be used. Fuel assemblies shall be limited to those fuel designs that have been analyzed with applicable NRC staff approved codes and methods and shown by tests or analyses to comply with all fuel safety design bases. A limited number of lead test assemblies that have not completed representative testing may be placed in nonlimiting core regions.

5.2.2 CONTROL ROD ASSEMBLIES

The reactor core shall contain 48 full length and no part length control rod assemblies. The full length control rod assemblies shall contain a nominal 142 inches of absorber material. The nominal values of absorber material shall be 80 percent silver, 15 percent indium and 5 percent cadmium. All control rods shall be clad with stainless steel tubing.

5.3 FUEL STORAGE

5.3.1 CRITICALITY

5.3.1.1 The spent fuel storage racks are designed and shall be maintained with:

a. Fuel assemblies having a maximum U-235 enrichment as set forth in Specification 3.9.14;

b. $K_{eff} \leq 0.95$ if fully flooded with unborated water, which includes an allowance for uncertainties as described in UFSAR Section 9.12;

c. A nominal center to center distance between fuel assemblies placed in the fuel storage racks of 10.82 inch for Region 1, with 9.02 inch for Regions 2 and 3;

d. Fuel assembly storage shall comply with the requirements of Specification 3.9.14.

5.3.1.2 The new fuel storage racks are designed and shall be maintained with:

a. Fuel assemblies having a maximum U-235 enrichment of 4.5 weight percent;

b. $K_{eff} \leq 0.95$ if fully flooded with unborated water, which includes an allowance for uncertainties as described in UFSAR Section 9.12;

c. $K_{eff} \leq 0.95$ if moderated by aqueous foam, which includes an allowance for uncertainties as described in UFSAR Section 9.12;

d. A nominal 21 inch center to center distance between fuel assemblies placed in the storage racks.

5.3.2 DRAINAGE

The spent fuel storage pool is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 750' - 10".

5.3.3 CAPACITY

The fuel storage pool is designed and shall be maintained with a storage capacity limited to no more than 1627 fuel assemblies.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.83
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated September 9, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-73 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 83 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DLCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

   In addition, the license is amended to add paragraph 2.C.(11) to the Facility Operating License No. NPF-73 as follows:

   (11) Additional Conditions

   The Additional Conditions contained in Appendix D, as revised through Amendment No. 83 , are hereby incorporated into this license. Duquesne Light Company shall operate the facility in accordance with the Additional Conditions.

3. This license amendment is effective as of its date of issuance and the change to the facility shall be implemented within 60 days from the date of issuance. Implementation of this amendment shall include the relocation of certain technical specification requirements to the appropriate licensee-controlled documents as described in the Licensee's application dated September 9, 1996, and evaluated in the staff's safety evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION

Chester Poslusny for

John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments: 1. Page 5 to License* No. NPF-73 and page 1 to Appendix D of License* No. NPF-73
2. Changes to the Technical Specifications

Date of Issuance: April 14, 1997

* Page 5 of the license and page 1 of Appendix D are attached, for convenience, for the composite license to reflect this change.

ATTACHMENT TO LICENSE AMENDMENT NO. 83

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

1. Revise License as follows:

Remove Page	Insert Page
5	5

2. Insert page 1 of Appendix D of the License

3. Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	Insert
XIII	XIII
XIV	XIV
XV	XV
XVI	XVI
5-1	5-1
5-2	-
5-3	-
5-4	-
5-5	-
5-6	-
5-7	5-2

(8) Detailed Control Room Design Review (DCRDR)

Deleted

(9) Safety Parameter Display System (SPDS)

Deleted

(10) Fire Protection Modifications (Section 9.5.1 of SER Supplement 6)

Deleted

(11) Additional Conditions

The Additional Conditions contained in Appendix D, as revised through Amendment No. 83 , are hereby incorporated into this licensee. Duquesne Light Company shall operate the facility in accordance with the Additional Conditions.

D. Exemptions

The following exemptions are authorized by law and will not endanger life or property or the common defense and security, and certain special circumstances are present. With the granting of these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

(1) The facility requires an exemption from the requirements of General Design Criterion (GDC) 4, Appendix A to 10 CFR 50. The staff has described in detail in Supplement 4 and Supplement 5 to the Safety Evaluation Report the technical basis and "special circumstances" associated with this exemption. The staff's environmental assessment was published on March 27, 1987 (52 FR 9979). Therefore, pursuant to 10 CFR 50.12(a)(1), 10 CFR 50.12(a)(2)(ii) and (iv), Beaver Valley Power Station, Unit 2 is exempt from the requirements of GDC 4, Appendix A to 10 CFR 50 with respect to the dynamic loading effects associated with the postulated pipe breaks described in detail in Section 3.6.3 of Supplement 4 to the Safety Evaluation Report. These dynamic loading effects include pipe whip, jet impingement, and break-associated dynamic transients. Specifically, this eliminates the need to install jet impingement barriers and pipe whip restraints associated with postulated pipe breaks in the pressurizer surge line, reactor coolant bypass system,

APPENDIX D

ADDITIONAL CONDITIONS
OPERATING LICENSE NO. NPF-73

Duquesne Light Company, Ohio Edison Company, The Cleveland Electric Illuminating Company, and The Toledo Edison Company shall comply with the following conditions on the schedules noted below:

Amendment Number	Additional Condition	Implementation Date
83	The licensee is authorized to relocate certain Technical Specification requirements to licensee-controlled documents. Implementation of this amendment shall include the relocation of these technical specification requirements to the appropriate documents, as described in the licensee's application dated September 9, 1996, and evaluated in the staff's safety evaluation attached to this amendment.	The amendment shall be implemented within 60 days from April 14, 1997

INDEX

BASES

SECTION		PAGE
3/4.9.12 AND 3/4.9.13	FUEL BUILDING VENTILATION SYSTEM	B 3/4 9-3
3/4.9.14	FUEL STORAGE - SPENT FUEL STORAGE POOL ..	B 3/4 9-4
3/4.10 SPECIAL TEST EXCEPTIONS		
3/4.10.1	SHUTDOWN MARGIN	B 3/4 10-1
3/4.10.2	GROUP HEIGHT, INSERTION AND POWER DISTRIBUTION LIMITS	B 3/4 10-1
3/4.10.3	PHYSICS TESTS	B 3/4 10-1
3/4.10.4	REACTOR COOLANT LOOPS	B 3/4 10-1
3/4.10.5	POSITION INDICATION SYSTEM-SHUTDOWN	B 3/4 10-1
3/4.11 RADIOACTIVE EFFLUENTS		
3/4.11.1	LIQUID EFFLUENTS	B 3/4 11-1
3/4.11.2	GASEOUS EFFLUENTS	B 3/4 11-1

DESIGN FEATURES

SECTION	PAGE
5.1 SITE LOCATION	5-1
5.2 REACTOR CORE	5-1
5.3 FUEL STORAGE	5-1

ADMINISTRATIVE CONTROLS

SECTION		PAGE
6.1 RESPONSIBILITY		6-1
6.2 ORGANIZATION		
6.2.1	ONSITE AND OFFSITE ORGANIZATIONS	6-1

INDEX

ADMINISTRATIVE CONTROLS

SECTION		PAGE
6.2.2	UNIT STAFF	6-1
6.2.3	INDEPENDENT SAFETY EVALUATION GROUP (ISEG)	6-2
6.3 FACILITY STAFF QUALIFICATIONS		6-6
6.4 TRAINING		6-6
6.5 REVIEW AND AUDIT		
6.5.1	ONSITE SAFETY COMMITTEE (OSC)	
6.5.1.1	Function	6-6
6.5.1.2	Composition	6-6
6.5.1.3	Alternates	6-7
6.5.1.4	Meeting Frequency	6-7
6.5.1.5	Quorum	6-7
6.5.1.6	Responsibilities	6-7
6.5.1.7	Authority	6-8
6.5.1.8	Records	6-8
6.5.2	OFFSITE REVIEW COMMITTEE (ORC)	
6.5.2.1	Function	6-8
6.5.2.2	Composition	6-9
6.5.2.3	Alternates	6-9
6.5.2.4	Consultants	6-9
6.5.2.5	Meeting Frequency	6-9
6.5.2.6	Quorum	6-9
6.5.2.7	Review	6-9

INDEX

ADMINISTRATIVE CONTROLS

SECTION		PAGE
6.5.2.8	Audits	6-10
6.5.2.9	Authority	6-11
6.5.2.10	Records	6-11
6.6 REPORTABLE EVENT ACTION		6-11
6.7 SAFETY LIMIT VIOLATION		6-12
6.8 PROCEDURES		6-12
6.9 REPORTING REQUIREMENTS		
6.9.1	ROUTINE REPORTS	6-16
6.9.1.1	Startup Reports	6-16
6.9.1.4	Annual Reports	6-17
6.9.1.6	Monthly Operating Report	6-18
6.9.1.10	Annual Radiological Environmental Operating Report	6-19
6.9.1.11	Annual Radioactive Effluent Release Report	6-19
6.9.1.12	Core Operating Limits Report	6-19
6.9.2	SPECIAL REPORTS	6-20
6.10 RECORD RETENTION		6-21
6.11 RADIATION PROTECTION PROGRAM		6-23
6.12 HIGH RADIATION AREA		6-23
6.13 PROCESS CONTROL PROGRAM (PCP)		6-24

INDEX

ADMINISTRATIVE CONTROLS

SECTION	PAGE
6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)	6-25
6.16 MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS (Liquid, Gaseous and Solid)	6-25
6.17 CONTAINMENT LEAKAGE RATE TESTING PROGRAM	6-25

5.1 SITE LOCATION

The Beaver Valley Power Station Unit No. 2 is located in Shippingport Borough, Beaver County, Pennsylvania, on the south bank of the Ohio River. The site is approximately 1 mile southeast of Midland, Pennsylvania, 5 miles east of East Liverpool, Ohio, and approximately 25 miles northwest of Pittsburgh, Pennsylvania. The exclusion area boundary has a minimum radius of 2000 feet around the Unit No. 1 containment building.

5.2 REACTOR CORE

5.2.1 FUEL ASSEMBLIES

The reactor shall contain 157 fuel assemblies. Each assembly shall consist of a matrix of Zircaloy or ZIRLO fuel rods with an initial composition of natural or slightly enriched uranium dioxide (UO_2) as fuel material. Limited substitutions of zirconium alloy or stainless steel filler rods for fuel rods, in accordance with approved applications of fuel rod configurations, may be used. Fuel assemblies shall be limited to those fuel designs that have been analyzed with applicable NRC staff approved codes and methods and shown by tests or analyses to comply with all fuel safety design bases. A limited number of lead test assemblies that have not completed representative testing may be placed in nonlimiting core regions.

5.2.2 CONTROL ROD ASSEMBLIES

The reactor core shall contain 48 full length and no part length control rod assemblies. The full length control rod assemblies shall contain a nominal 142 inches of absorber material. The nominal values of absorber material shall be 80 percent silver, 15 percent indium and 5 percent cadmium. All control rods shall be clad with stainless steel tubing.

5.3 FUEL STORAGE

5.3.1 CRITICALITY

5.3.1.1 The spent fuel storage racks are designed and shall be maintained with:

a. Fuel assemblies having a maximum U-235 enrichment as set forth in Specification 3.9.14;

b. $K_{eff} \leq 0.95$ if fully flooded with unborated water, which includes an allowance for uncertainties as described in UFSAR Section 9.1;

c. A minimum center to center distance between fuel assemblies placed in the fuel storage racks of 10.4375 inches;

d. Fuel assembly storage shall comply with the requirements of Specification 3.9.14.

5.3.1.2 The new fuel storage racks are designed and shall be maintained with:

a. Fuel assemblies having a maximum U-235 enrichment of 4.85 weight percent;

b. $K_{eff} \leq 0.95$ if fully flooded with unborated water, which includes an allowance for uncertainties as described in UFSAR Section 9.1;

c. $K_{eff} \leq 0.95$ if moderated by aqueous foam, which includes an allowance for uncertainties as described in UFSAR Section 9.1;

d. A nominal 21 inch center to center distance between fuel assemblies placed in the storage racks.

5.3.2 DRAINAGE

The spent fuel storage pool is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 751'-3".

5.3.3 CAPACITY

The fuel storage pool is designed and shall be maintained with a storage capacity limited to no more than 1088 fuel assemblies.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 202 AND 83 TO FACILITY OPERATING

LICENSE NOS. DPR-66 AND NPF-73

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-334 AND 50-412

1.0 INTRODUCTION

By letter dated September 9, 1996, the Duquesne Light Company (the licensee) submitted a request for changes to the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and BVPS-2) Technical Specifications (TSs). The requested changes would modify the design features section (Section 5.0) of the TSs to make the design features section consistent with the four criteria specified in 10 CFR 50.36 and with the guidance provided in the NRC's Standard Technical Specifications, Westinghouse Plants (NUREG-1431, Revision 1).

2.0 EVALUATION

The proposed change to the design features section (Section 5.0) of the TSs would delete certain requirements from the current TSs and add some other requirements not presently included in the TSs. The proposed changes to the TSs would include some minor editorial changes and would correct the TS Index pages. The proposed editorial changes and the Index pages are acceptable since they do not change the intent of any TSs nor do they change any TS requirements. The proposed deletions and additions are evaluated below.

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36. That regulation requires that the TSs include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation;

9704230249 970414
PDR ADOCK 05000334
P PDR

(3) surveillance requirements; (4) design features; and (5) administrative controls.

The four criteria defined by 10 CFR 50.36 for determining whether particular limiting conditions for operation are required to be included in the TSs, are as follows: (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety.

Existing TS limiting conditions for operation which do not satisfy these four specified criteria may be relocated to the Updated Final Safety Analysis Report (UFSAR), such that future changes could be made to these provisions pursuant to 10 CFR 50.59. Other requirements may be relocated to more appropriate documents (e.g. Security Plan, Quality Assurance (QA) plan, and Emergency Plan) and controlled by the applicable regulatory requirement. While the content of the TSs design features and administrative controls are specified in 10 CFR 50.36(c)(4) and 10 CFR 50.36(c)(5), respectively, particular details of the design features and administrative controls may be relocated to licensee-controlled documents where 10 CFR 50.59 or comparable regulatory controls exist.

With respect to design features, 10 CFR 50.36(c)(4) requires that design features to be included in TSs are those features of the facility such as materials of construction and geometric arrangements, which, if altered or modified, would have a significant effect on safety and are not covered as TS safety limits, limiting safety system settings, limiting conditions for operation, or surveillance requirements.

Based on the guidance provided in 10 CFR 50.36 and in NUREG-1431, Revision 1, the licensee has proposed the following TS changes, deletions, and additions:

1. TS 5.1 (Site) would be modified to be consistent with the guidance provided in NUREG-1431, Revision 1. TS 5.1 would be re-titled to Site Location and consistent with NUREG-1431, Revision 1, would include a text description of the site rather than referencing and including BVPS-1 TS Figures 5.1-1 through 5.1-6 and BVPS-2 TS Figures 5.1-1 through 5.1-4.

   The site boundary, shown in TS Figure 5.1-1 for both units, is also shown in Figure 5.1-1 of the offsite dose calculation manual (ODCM).

Reference to BVPS-1 TS Figure 5.1-2 in TS 5.1.2 would be deleted since BVPS-1 TS Figure 5.1-2 was previously deleted by License Amendment No. 121. Therefore, deletion of reference to BVPS-1 TS Figure 5.1-2 in TS 5.1.2 is acceptable.

The exclusion area boundary, shown on TS Figure 5.1-3 for both units, is also described and shown in BVPS-1 Updated Final Safety Analysis Report (UFSAR) Section 2.1.1 and BVPS-1 UFSAR Figure 2.1-7 and in BVPS-2 UFSAR Section 2.1.2.1 and BPVS-2 UFSAR Figure 2.1-2.

The low population zone shown on TS Figure 5.1-4 for both units, is also described in BVPS-1 UFSAR Section 2.1.2 and in BVPS-2 UFSAR Section 2.1.3.5.

The gaseous and liquid release points shown in BVPS-1 TS Figures 5.1-5 and 5.1-6 and in BVPS-2 TS Figure 5.1-2 are also shown in ODCM Figure 2.4-2 for the gaseous release points and ODCM Figures 1.4-1 and 1.4-2 for the BVPS-1 and BVPS-2 liquid release points.

The flood control requirements of TS 5.1.5 are also described in BVPS-1 UFSAR Section 2.3.3 and in BVPS-2 UFSAR Section 3.4.1.

The requirements of 10 CFR 50.36(c)(4) do not require the inclusion of figures showing site arrangements. However, 10 CFR 50.36(c)(4) does require the inclusion of geometric arrangements. The proposed change to TS 5.1 provides a text description, which meets this criteria. A text description of the site is consistent with the guidance provided in NUREG-1431, Revision 1. The TS figures are also shown in UFSAR and ODCM figures. Any changes to UFSAR figures will be subject to the requirements of 10 CFR 50.59, while any changes to ODCM figures will be subject to the change requirements of TS 6.14. Therefore, the proposed changes to TS 5.1 are acceptable.

2. TS 5.2 (Containment) would be deleted. The requirements contained in TS 5.2 are contained in BVPS-1 UFSAR Sections 5.2.1, 5.2.2, 5.2.4, and Table 14.3.3a and in BVPS-2 UFSAR Sections 3.8.1, 6.2.1, 6.2.4, and Table 6.2-3. Details regarding containment leakage rates, containment air lock leakage rates, and containment structural integrity are retained in TSs 3/4.6.1.2, 3/4.6.1.3, and 3/4.6.1.6 to ensure any changes that may significantly affect safety would require prior NRC review and approval. Design features, which are being relocated from the TSs, would not have a significant effect on safety if altered or modified and are not otherwise required to be in the TSs pursuant to 10 CFR 50.36(c). Any changes to the UFSAR will be subject to the requirements of 10 CFR 50.59. Therefore, the proposed deletion of TS 5.2 is acceptable.

3. TS 5.3 (Reactor Core) would be renumbered to TS 5.2. The text for renumbered TS 5.2.1 (Fuel Assemblies) and TS 5.2.2 (Control Rod Assemblies) would not be changed for either unit. The proposed changes are administrative only and are, therefore, acceptable.

4. TS 5.4 (Reactor Coolant System) would be deleted. The requirements contained in TS 5.4 are also contained in BVPS-1 UFSAR Section 4.1 and 4.2 and in BVPS-2 UFSAR Sections 5.2, 5.3, and 5.4. Details regarding reactor coolant system leakage and structural integrity are retained in TSs 3/4.4.6.2 and 3/4.4.10 to ensure any changes that may significantly affect safety would require prior NRC review and approval. Design features, which are being relocated from the TSs, would not have a significant effect on safety if altered or modified and are not otherwise required to be in the TSs pursuant to 10 CFR 50.36(c). Any changes to the UFSAR will be subject to the requirements of 10 CFR 50.59. Therefore, the proposed deletion of TS 5.4 is acceptable.

5. TS 5.5 (Emergency Core Cooling Systems) would be deleted. The requirements contained in TS 5.5 are also contained in UFSAR Section 6.3 for both units. Details of the design features and operability requirements for the emergency core cooling system are retained in TSs 3/4.5.1, 3/4.5.2, 3/4.5.3, 3/4.5.4.1.1, 3/4.5.4.1.2, and 3/4.5.5 to ensure any changes that may significantly affect safety would require prior NRC review and approval. Design features, which are being relocated from the TSs, would not have a significant effect on safety if altered or modified and are not otherwise required to be in the TSs pursuant to 10 CFR 50.36(c). Any changes to the UFSAR will be subject to the requirements of 10 CFR 50.59. Therefore, the proposed deletion of TS 5.5 is acceptable.

6. TS 5.6 (Fuel Storage) would be renumbered to TS 5.3. The text for renumbered TS 5.3.1 (Criticality) would be replaced with TS 5.3.1.1 for the spent fuel storage racks and with TS 5.3.1.2 for the new fuel storage racks. The proposed revision to renumbered TS 5.3.1 would make its format consistent with NUREG-1431, Revision 1; TS 5.3.1.1 would contain the design requirements for the spent fuel storage racks and TS 5.3.1.2 would contain the design requirements for the new fuel storage racks.

   Proposed TS 5.3.1.1.a would specify a maximum U-235 enrichment of spent fuel assemblies in the spent fuel racks as set forth in TS 3.9.14. TS 3.9.14 specifies storage of spent fuel in the various regions of the spent fuel storage pool as a function of original enrichment percentage and fuel burnup. Although proposed TS 5.3.1.1.a does not specify a single enrichment value, as does NUREG-1431, Revision 1, proposed TS 5.3.1.1.a is acceptable since it is consistent with TS 3.9.14 which was issued as License Amendment No. 178 for BVPS-1 and as License Amendment No. 12 for BVPS-2.

   Proposed TS 5.3.1.1.b would require the spent fuel storage racks be designed and maintained with $K_{eff} \leq 0.95$ if flooded with unborated water including an allowance for uncertainties. This requirement is equivalent to current TS 5.6.1. Therefore, proposed TS 5.3.1.1.b is acceptable.

Proposed TS 5.3.1.1.c specifies the spacing of fuel assemblies in storage racks. The proposed TS provides the same spacings as in part of current TS 5.6.1; therefore, proposed TS 5.3.1.1.c is acceptable.

Proposed TS 5.3.1.1.d would require that fuel assembly storage shall comply with the requirements of TS 3.9.14. This is acceptable since TS 3.9.14 provides adequate requirements for spent fuel storage.

Proposed TS 5.3.1.2.a would specify a maximum U-235 enrichment of 4.5 weight percent for BVPS-1 and 4.85 weight percent for BVPS-2 new fuel assemblies in the new fuel storage racks. These maximum enrichment values are acceptable since they are consistent with the maximum enrichment values and criticality analyses approved in License Amendment No. 114 for BVPS-1 and License Amendment No. 12 for BVPS-2.

Proposed TS 5.3.1.2.b would require the new fuel storage racks be designed and maintained with $K_{eff} \leq 0.95$ if fully flooded with unborated water including an allowance for uncertainties. This requirement is consistent with the criteria provided in BVPS-1 UFSAR Section 9.12 and BVPS-2 UFSAR Section 9.1 and in the NRC's Standard Review Plan (NUREG-0800) Section 9.1.1. Therefore, proposed TS 5.3.1.2.b is acceptable.

Proposed TS 5.3.1.2.c would require the new fuel storage racks be designed and maintained with $k_{eff} \leq 0.95$ if moderated by aqueous foam (low density optimum-moderation conditions) including an allowance for uncertainties. This requirement is consistent with the criteria provided in BVPS-1 UFSAR Section 9.12 and BVPS-2 UFSAR Section 9.1 and in the NRC's Standard Review Plan (NUREG-0800) Section 9.1.1. Therefore, proposed TS 5.3.1.2.c is acceptable.

Proposed TS 5.3.1.2.d would require the new fuel storage racks be designed and maintained to provide a nominal 21-inch center-to-center spacing between fuel assemblies placed in the storage racks. This requirement ensures that k_{eff} is maintained ≤ 0.95 (in accordance with the requirements of the NRC's Standard Review Plan (NUREG-0800) Section 9.1.1) and is consistent with the criteria provided in BVPS-1 UFSAR Section 9.12 and BVPS-2 UFSAR Section 9.1. Therefore, proposed TS 5.3.1.2.d is acceptable.

7. TS 5.6.2 (Drainage) and TS 5.6.3 (Capacity) would be renumbered to TS 5.3.2 and TS 5.3.3, respectively; the text for these TSs would not be changed for either unit. The proposed changes are administrative only and are, therefore, acceptable.

8. TS 5.7 (Seismic Classification) and TS 5.8 (Meteorological Tower Location) would be deleted from the TSs. The requirements contained in TS 5.7 are also contained in BVPS-1 UFSAR Appendix B and BVPS-2 UFSAR Section 3.7. The location of the meteorological tower is also provided in ODCM Figure 5-1. The requirements of 10 CFR 50.36(c)(4) do not require the inclusion of the seismic classification of structures, systems, and components nor the inclusion of the meteorological tower

location in TSs. The seismic classification of structures, systems, and components is adequately specified in the UFSARs and the meteorological tower location is adequately specified in the ODCM. Any changes to the UFSARs will be subject to the requirements of 10 CFR 50.59 and any changes to the ODCM will be subject to the requirements of TS 6.14. Therefore, the proposed deletion of TSs 5.7 and 5.8 is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 64384). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Donald S. Brinkman

Date: April 14, 1997