



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 10, 1999

Mr. J. E. Cross
President-Generation Group
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENTS RE: REQUIRED QUALIFICATIONS FOR OPERATIONS MANAGEMENT AND USE OF GENERIC POSITION TITLES IN TECHNICAL SPECIFICATIONS (TAC NOS. MA5059 AND MA5060)

Dear Mr. Cross:

The Commission has issued the enclosed Amendment No. 224 to Facility Operating License No. DPR-66 and Amendment No. 100 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) and Updated Final Safety Analysis Report (UFSAR) in response to your application dated March 3, 1999, as supplemented May 27, 1999, and June 22, 1999, which submitted License Amendment Request Nos. 259 and 131.

These amendments change the required qualifications for operations management specified in the TSs for the Beaver Valley Power Station, Units 1 and 2 (BVPS-1 and BVPS-2). The requirement that the operations manager hold a Senior Reactor Operator (SRO) license at the time of appointment is changed in the TSs to require that the assistant operations managers, one for each unit, hold an SRO license on their assigned unit. The revised TSs require the operations manager to hold, or have held, an SRO license. Additionally, the UFSAR for each unit is changed to require the operations manager to "hold, or have held," an SRO license rather than "hold" a license. The revised UFSARs require the same as the TSs; that the assistant operations managers hold an SRO license on the unit to which they are assigned. Finally, the amendments substitute generic personnel titles for plant-specific personnel titles in the BVPS-1 and BVPS-2 TSs. The correlation between generic titles and plant-specific titles is provided in the revised BVPS-2 UFSAR.

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August 10, 1999

Mr. J. E. Cross

- 2 -

A copy of our safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Daniel S. Collins, Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

- Enclosures: 1. Amendment No. 224 to DPR-66
- 2. Amendment No. 100 to NPF-73
- 3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION:

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Official Record Copy

August 10, 1999

Mr. J. E. Cross

- 2 -

A copy of our safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Daniel S. Collins, Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

- Enclosures: 1. Amendment No. 224 to DPR-66
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DATE	7-16-99		7/16/99	6/13/99	7/30/99	8/9/99

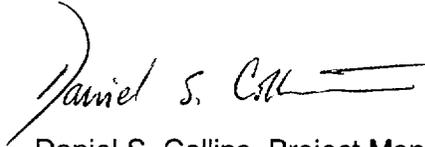
Official Record Copy

Mr. J. E. Cross

- 2 -

A copy of our safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Daniel S. Collins". The signature is written in black ink and is positioned above the typed name.

Daniel S. Collins, Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosures: 1. Amendment No. 224 to DPR-66
2. Amendment No. 100 to NPF-73
3. Safety Evaluation

cc w/encls: See next page

Beaver Valley Power Station, Units 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 224
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated March 3, 1999, as supplemented May 27, 1999, and June 22, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 224, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 10, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 224

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

6 - 1
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Insert

6 - 1
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6.1 RESPONSIBILITY

6.1.1 The plant manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

6.2.1 ONSITE AND OFFSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for facility operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements, including the plant-specific titles of those personnel fulfilling the responsibilities of the positions delineated in these technical specifications shall be documented in the Unit 2 UFSAR. The correlation between the positions described in these technical specifications and the plant-specific titles are documented in the Unit 2 UFSAR, Table 13.1-2.
- b. The plant manager shall be responsible for overall safe operation of the plant and shall have control over those onsite activities necessary for safe operation and maintenance.
- c. The corporate officer with direct responsibility for the plant shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out radiation protection and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

UNIT STAFF (Continued)

- c. A break of at least eight hours should be allowed between work periods, including shift turnover time.
- d. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the plant manager or predesignated alternate, or higher levels of management. Authorized deviations to the working hour guidelines shall be documented and available for NRC review.

- g. The operations manager shall either hold a senior reactor operator license or have held a senior reactor operator license for a pressurized water reactor. The assistant operations manager shall hold a current senior reactor operator license.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility and radiation protection staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the operations manager as specified in Specification 6.2.2.g, the radiation protection manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the technical advisory engineering representative who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.

6.5 DELETED

6.6 REPORTABLE EVENT ACTION

- 6.6.1 The following actions shall be taken for REPORTABLE EVENTS:
- a. The Commission shall be notified in accordance with 10 CFR 50.72 and/or a report be submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
 - b. Each REPORTABLE EVENT shall be reviewed by the OSC, and the results of this review shall be submitted to the ORC.

6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:
- a. The facility shall be placed in at least HOT STANDBY within one (1) hour.
 - b. The Safety Limit violation shall be reported to the Commission within one hour and to the plant manager and to the ORC within 24 hours.
 - c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the On-Site Safety Committee (OSC). This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

SAFETY LIMIT VIOLATION (Continued)

- d. The Safety Limit Violation Report shall be submitted to the Commission, the ORC and the plant manager within 30 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Not used.
- e. Not used.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.

6.8.2 Deleted

6.8.3 Deleted

6.8.4 A Post-Accident monitoring program shall be established, implemented, and maintained:

A program which will provide the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples following an accident. The program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for sampling and analysis, and
- (iii) Provisions for maintenance of sampling and analysis equipment.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.1601 of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiological Work Permit⁽¹⁾. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
- c. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by a facility radiation protection supervisor in the Radiological Work Permit.

6.12.2 The requirements of 6.12.1, above, also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the shift supervisor on duty and/or a facility radiation protection supervisor.

(1) Radiation protection personnel, or personnel escorted by radiation protection personnel in accordance with approved emergency procedures, shall be exempt from the RWP issuance requirement during the performance of their radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

6.13 PROCESS CONTROL PROGRAM (PCP)

Changes to the PCP:

- a. Shall be documented and records of reviews performed shall be retained in accordance with the applicable record retention provision of the quality assurance program description referenced in the Updated Final Safety Analysis Report. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - 2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
- b. Shall become effective after review and acceptance by the OSC and the approval of the plant manager, predesignated alternate or a predesignated manager to whom the plant manager has assigned in writing the responsibility for review and approval of specific subjects.

6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained in accordance with the applicable record retention provision of the quality assurance program description referenced in the Updated Final Safety Analysis Report. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.
- b. Shall become effective after review and acceptance by the OSC and the approval of the plant manager, predesignated alternate or a predesignated manager to whom the plant manager has assigned in writing the responsibility for review and approval of specific subjects.

OFFSITE DOSE CALCULATION MANUAL (ODCM) (Continued)

- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

6.16 Moved to the PROCESS CONTROL PROGRAM.

6.17 Containment Leakage Rate Testing Program

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions⁽¹⁾. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995.

The peak calculated containment internal pressure for the design basis loss of coolant accident, P_a , is 40.0 psig.

The maximum allowable containment leakage rate, L_a , at P_a , shall be 0.10% of containment air weight per day.

Leakage Rate acceptance criteria are:

- a. Containment leakage rate acceptance criterion is $\leq 1.0 L_a$ for the overall Type A leakage test and $< 0.60 L_a$ for the Type B and Type C tests on a minimum pathway leakage rate (MNPLR) basis. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $< 0.60 L_a$ on a maximum pathway leakage rate (MXPLR)⁽²⁾ basis for Type B and Type C tests and $< 0.75 L_a$ for Type A tests.

(1) Exemptions to Appendix J of 10 CFR 50 dated November 19, 1984, December 5, 1984, and July 26, 1995.

(2) For penetrations which are isolated by use of a closed valve(s), blind flange(s), or de-activated automatic valve(s), the MXPLR of the isolated penetration is assumed to be the measured leakage through the isolation device(s).



UNITED STATES
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WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 100
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated March 3, 1999, as supplemented May 27, 1999, and June 22, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 100 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DLCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 10, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 100

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

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ADMINISTRATIVE CONTROLS

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6.1 RESPONSIBILITY

6.1.1 The plant manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION6.2.1 ONSITE AND OFFSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for facility operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements, including the plant-specific titles of those personnel fulfilling the responsibilities of the positions delineated in these technical specifications shall be documented in the Unit 2 UFSAR. The correlation between the positions described in these technical specifications and the plant-specific titles are documented in the Unit 2 UFSAR, Table 13.1-2.
- b. The plant manager shall be responsible for overall safe operation of the plant and shall have control over those onsite activities necessary for safe operation and maintenance.
- c. The corporate officer with direct responsibility for the plant shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out radiation protection and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 UNIT STAFF

The unit organization shall be subject to the following:

- a. Each duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed operators shall be in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be onsite when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed senior reactor operator or senior reactor operator limited to fuel handling who has no other concurrent responsibilities during this operation.
- f. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions; senior reactor operators, reactor operators, radiation control technicians, auxiliary operators, meter and control repairman, and all personnel actually performing work on safety related equipment.

The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the plant is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance or major plant modifications, on a temporary basis, the following guidelines shall be followed:

- a. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
- b. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any seven day period, all excluding shift turnover time.
- c. A break of at least eight hours should be allowed between work periods, including shift turnover time.

UNIT STAFF (Continued)

- d. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the plant manager or predesignated alternate, or higher levels of management. Authorized deviations at the working hour guidelines shall be documented and available for NRC review.

- g. The operations manager shall either hold a senior reactor operator license or have held a senior reactor operator license for a pressurized water reactor. The assistant operations manager shall hold a current senior reactor operator license.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility and radiation protection staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the operations manager as specified in Specification 6.2.2.g, the radiation protection manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the technical advisory engineering representative who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.

6.5 DELETED

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified in accordance with 10 CFR 50.72 and/or a report be submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the OSC, and the results of this review shall be submitted to the ORC.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one (1) hour.
- b. The Safety Limit violation shall be reported to the Commission within one hour. The Safety Limit violation shall be reported to the plant manager and to the ORC within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the On-Site Safety Committee (OSC). This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

SAFETY LIMIT VIOLATION (Continued)

- d. The Safety Limit Violation Report shall be submitted to the Commission, the ORC and the plant manager within 30 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Not used.
- e. Not used.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.

6.8.2 Deleted

6.8.3 Deleted

6.8.4 A Post-Accident monitoring program shall be established, implemented, and maintained. The program will provide the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples following an accident. The program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for sampling and analysis, and
- (iii) Provisions for maintenance of sampling and analysis equipment.

HIGH RADIATION AREA (Continued)

issuance of a Radiological Work Permit⁽¹⁾. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
- c. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by a facility radiation protection supervisor in the Radiological Work Permit.

6.12.2 The requirements of 6.12.1, above, also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the shift supervisor on duty and/or a facility radiation protection supervisor.

(1) Radiation protection personnel, or personnel escorted by radiation protection personnel in accordance with approved emergency procedures, shall be exempt from the RWP issuance requirement during the performance of their radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

6.13 PROCESS CONTROL PROGRAM (PCP)

Changes to the PCP:

- a. Shall be documented and records of reviews performed shall be retained in accordance with the applicable record-retention provision of the quality assurance program description included in the Updated Final Safety Analysis Report. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - 2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
- b. Shall become effective after review and acceptance by the OSC and the approval of the plant manager, predesignated alternate or a predesignated manager to whom the plant manager has assigned in writing the responsibility for review and approval of specific subjects.

6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained in accordance with the applicable record retention provision of the quality assurance program description included in the Updated Final Safety Analysis Report. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.
- b. Shall become effective after review and acceptance by the OSC and the approval of the plant manager, predesignated alternate or a predesignated manager to whom the plant manager has assigned in writing the responsibility for review and approval of specific subjects.
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

6.16 Moved to the PROCESS CONTROL PROGRAM.

6.17 CONTAINMENT LEAKAGE RATE TESTING PROGRAM

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54 (c) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions⁽¹⁾. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995.

(1) Exemptions to Appendix J of 10 CFR 50, as stated in the operating license.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 224 AND 100 TO FACILITY OPERATING

LICENSE NOS. DPR-66 AND NPF-73

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-334 AND 50-412

1.0 INTRODUCTION

By letter dated March 3, 1999, as supplemented May 27, 1999, and June 22, 1999, Duquesne Light Company (DLC, the licensee) requested approval to amend the Technical Specifications (TSs) for the Beaver Valley Power Station, Units 1 and 2 (BVPS-1 and BVPS-2). The proposed amendment incorporated two changes: (1) modifying the licensee's commitment to American National Standards Institute (ANSI) N18.1-1971 for the operations manager and (2) incorporating generic titles in the technical specifications to eliminate the need to request an amendment whenever the plant-specific titles are changed. Following Nuclear Regulatory Commission (NRC) review and subsequent discussions with the licensee related to their initial submittal, DLC provided a revised submittal on May 27, 1999, as supplemented June 22, 1999. The information submitted on May 27, 1999, incorporated revised qualifications for the operations manager into the TSs and changed the generic title proposed to replace the plant-specific title of Senior Vice President, Nuclear Power Division. The revisions provided in the May 27, 1999, letter, as well as the June 22, 1999, letter, which provided the final TS pages, did not change the initial proposed no significant hazards consideration determination or expand the amendment beyond the scope of the initial notice.

2. EVALUATION

DLC's current commitment to ANSI N18.1-1971, "Selection and Training of Nuclear Power Plant Personnel," requires the operations manager to hold an SRO license. The proposed amendment would add item (g) to TS 6.2.1, "Onsite and Offsite Organizations," which allows the operations manager, in lieu of meeting the N18.1 requirement of holding an SRO license, to (1) hold an SRO license or (2) have held a SRO license for a pressurized water reactor. This

approach is supported by ANSI/ANS 3.1-1987 and 3.1-1993, "Selection, Qualification and Training of Personnel for Nuclear Power Plants," which allow individuals to hold the position of operations manager based on the options proposed by DLC for TS 6.2.1(g). The 1987 and 1993 standards further state that if the operations manager does not hold a license then the operations middle manager shall hold an SRO license. The change to TS 6.2.1(g) proposed by the licensee, includes the requirement that the position of assistant operations manager hold a current SRO license.

In its letter of March 3, 1999, the licensee stated that the operations manager has overall responsibility for the operations of both units. The assistant operations manager at each unit is responsible for the direct oversight of the licensed operators and is required to hold an SRO license for the unit managed. The requirement that the assistant operations managers hold a unit-specific SRO license is consistent with the requirements of 10 CFR 50.54(l) and ensures that a licensed off-shift senior operator will direct the licensed activities of the licensed operators. The requirement for the operations manager to hold or have held a license together with the requirement that the assistant operations manager hold a license is consistent with ANSI 3.1-1987 and 1993.

In their March submittal, DLC proposed that generic titles be incorporated into the TSs for both units. The licensee has stated that the proposed amendment would eliminate the need to request an amendment whenever plant-specific titles are changed. To support this request, the licensee modified Table 13.1-2, "Personnel Responsibilities and Qualifications," in the Unit 2 Updated Final Safety Analysis Report (UFSAR). The table has been changed to list the current plant-specific position title together with the generic title to be referenced in the TSs for both units. The generic titles of plant manager, radiation protection manager, operations manager, and assistant operations manager are consistent with the position titles in ANSI/ANS Standards N18.1-1971, 3.1-1987, and 3.1-1993 for workers with comparable responsibilities.

Following NRC review and subsequent discussions with the licensee related to the generic title to be used for the individual with corporate responsibility for nuclear safety, DLC submitted revised information on May 27, 1999. In their May submittal, the licensee proposed to incorporate "corporate officer with direct responsibility for the plant" into the TSs to identify the individual with overall responsibility for plant nuclear safety. The UFSAR indicates that the plant-specific position tasked with that responsibility is the President, Generation Group and Chief Nuclear Officer. The corporate officer selected is clearly responsible for nuclear activities without having ancillary responsibilities that might detract from attention to nuclear safety matters. The generic title and defined responsibilities meet the relevant criteria in NUREG-0800, Standard Review Plan, Section 13.1.1, "Management and Technical Support Organization."

2.1. Summary

The staff reviewed the licensee's submittals and concludes that the modifications to the TSs requiring the assistant operations manager to hold an SRO license is consistent with the requirements of 10 CFR 50.54(l) and ensures that a licensed off-shift senior operator will direct the licensed activities of the licensed operators. Further, requiring the operations manager to hold or have held an SRO license is consistent with the requirements of ANSI/ANS Standards N18.1-1971, 3.1-1987, and 3.1-1993 and ensures site-specific, detailed, and relevant technical and systems knowledge in a senior operations management position. Therefore, the staff concludes that the proposed changes to the BVPS-1 and BVPS-2 TSs, which modify the

commitment to ANSI N18.1-1971 for the operations manager, are consistent with and meet the intent of the relevant review criteria and are acceptable.

The staff further concludes that the licensee's use of generic titles is consistent with the titles in the ANSI/ANS standards and meets the relevant review criteria in NUREG-0800 and is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official asked if the amendment would allow the licensee to place an individual in the operations manager position who does not hold an SRO license, but who has substantial nuclear experience from the military, and take credit for the military experience as satisfying the requirement to hold or have held an SRO license. The NRC staff responded that military nuclear experience can not be substituted in lieu of the requirement for the operations manager to hold, or have held, an SRO license. While ANSI/ANS 3.1-1987, and 3.1-1993 do allow some credit to be taken for military experience toward required nuclear industry experience for management positions, the TS and UFSAR requirement for the operations manager to hold, or have held, an SRO license will not allow such credit to be taken - an SRO license is a specific NRC license, as defined in 10 CFR 55.4, and military experience/qualifications are not considered to be equivalents/substitutes in lieu of this license. Furthermore, DLC has not committed to ANSI/ANS 3.1-1987 or 3.1-1993 in their Quality Assurance plan and, thus, can not invoke the provisions of these standards to take credit for military experience without prior NRC approval. The State official had no further questions or comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change administrative requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (64 FR 19556). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Ashley

Date: August 10, 1999