

August 11, 1993

Docket Nos. 50-334
and 50-412

Mr. J. D. Sieber, Senior Vice President
and Chief Nuclear Officer
Nuclear Power Division
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077-0004

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Dear Mr. Sieber:

SUBJECT: ISSUANCE OF AMENDMENT NOS. 174 AND 54 TO FACILITY OPERATING
LICENSES DPR-66 AND NPF-73, BEAVER VALLEY POWER STATION, UNIT NOS. 1
AND 2, IN RESPONSE TO CHANGE REQUEST NOS. 194/59, CONTAINMENT
RECIRCULATION SPRAY SYSTEM (TAC NOS. M83755 AND M83756)

The Commission has issued the enclosed Amendment Nos. 174 and 54 to
Facility Operating License Nos. DPR-66 and NPF-73 respectively, for the Beaver
Valley Power Station, Unit Nos. 1 and 2. The amendments consist of changes to
the Technical Specifications (TS) in response to your application dated
November 16, 1992, as supplemented March 11, 1993.

The amendments revise the Appendix A TS relating to the containment
recirculation spray system. The amendments add TS coverage to address the
failure of more subsystems than are now covered in TS. In addition, certain
requirements that were only applicable to cycle 8 operation are deleted.

A copy of the related Safety Evaluation is also enclosed. The Notice of
Issuance will be included in the Commission's biweekly Federal Register
notice.

Sincerely,

Original signed by:
Gordon E. Edison, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 174 to DPR-66
2. Amendment No. 54 to NPF-73
3. Safety Evaluation

cc w/enclosures:
See next page

OFFICE	PDI-3:LA	PDI-3:PM	OGC	PDI-3:D	
NAME	SLittle ^{ms}	GEdison:dt	C Marco	WButler	
DATE	9/14/93	8/14/93	8/19/93	8/11/93	

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

August 11, 1993

Docket Nos. 50-334
and 50-412

Mr. J. D. Sieber, Senior Vice President
and Chief Nuclear Officer
Nuclear Power Division
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077-0004

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A copy of the related Safety Evaluation is also enclosed. The Notice of
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notice.

Sincerely,

A handwritten signature in black ink that reads "Gordon E. Edison".

Gordon E. Edison, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 174 to DPR-66
2. Amendment No. 54 to NPF-73
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. J. D. Sieber
Duquesne Light Company

Beaver Valley Power Station
Units 1 & 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 174
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated November 16, 1992, as supplemented March 11, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 174, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 11, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 174

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 6-7

3/4 6-13

3/4 6-14

B 3/4 6-2

Insert

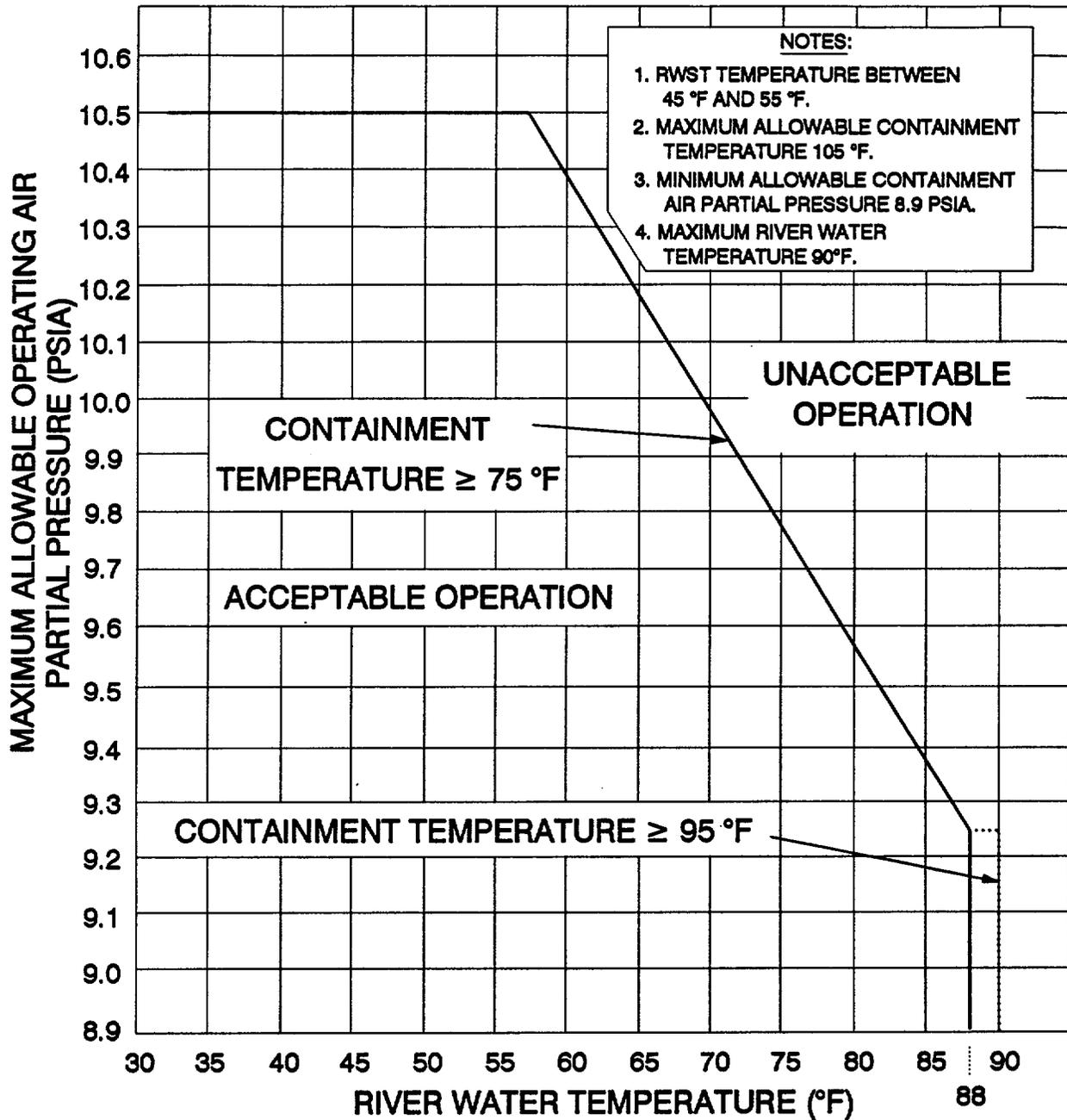
3/4 6-7

3/4 6-13

3/4 6-14

B 3/4 6-2

DPR-66



**FIGURE 3.6-1
MAXIMUM ALLOWABLE PRIMARY CONTAINMENT AIR PRESSURE
VERSUS RIVER WATER TEMPERATURE**

CONTAINMENT SYSTEMS

CONTAINMENT RECIRCULATION SPRAY SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.2.2 Four separate and independent containment recirculation spray subsystems, each composed of a spray pump, associated heat exchanger and flow path shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With one containment recirculation spray subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 7 days or be in HOT STANDBY within the next 6 hours; restore the inoperable spray system to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the next 30 hours.
- b. With two containment recirculation spray subsystems inoperable restore at least one inoperable subsystem to OPERABLE status within 72 hours or be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.2.2 Each containment recirculation spray subsystem shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each accessible valve (manual, power-operated, or automatic) in the flow path not locked, sealed or otherwise secured in position, is in its correct position;
- b. When tested pursuant to Specification 4.0.5, manually start each recirculation spray pump and verify the pump shaft rotates;
- c. At least once per 18 months by verifying that on a Containment Pressure-High-High signal, the recirculation spray pumps start automatically as follows:

RS-P-1A and RS-P-2B	210 ± 5 second delay
RS-P-2A and RS-P-1B	225 ± 5 second delay

CONTAINMENT SYSTEMSSURVEILLANCE REQUIREMENTS (continued)

- d. At least once per 18 months during shutdown, verify that on recirculation flow, each pump develops the required differential pressure and flow rate as shown below when tested pursuant to Specification 4.0.5:

RS-P-1A and RS-P-1B	≥ 127 psid at ≥ 2000 gpm
RS-P-2A and RS-P-2B	≥ 132 psid at ≥ 2000 gpm

- e. At least once per 18 months during shutdown, by:
1. Cycling each power operated (excluding automatic) valve in the flow path not testable during plant operation, through at least one complete cycle of full travel.
 2. Verifying that each automatic valve in the flow path actuates to its correct position on a test signal.
 3. Initiating flow through each River Water subsystem and its two associated recirculation spray heat exchangers, and verifying a flow rate of at least 8000 gpm.
- f. At least once per 5 years by performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed.

DPR-66
CONTAINMENT SYSTEMS

BASES

3/4.6.1.4 and 3/4.6.1.5 INTERNAL PRESSURE AND AIR TEMPERATURE

The limitations on containment internal pressure and average air temperature as a function of river water temperature ensure that 1) the containment structure is prevented from exceeding its design negative pressure of 8.0 psia, 2) the containment peak pressure does not exceed the design pressure of 45 psig during LOCA conditions, and 3) the containment pressure is returned to subatmospheric conditions following a LOCA.

The containment internal pressure and temperature limits shown as a function of river water temperature describe the operational envelope that will 1) limit the containment peak pressure to less than its design value of 45 psig and 2) ensure the containment internal pressure returns subatmospheric within 60 minutes following a LOCA.

The limits on the parameters of Figure 3.6-1 are consistent with the assumptions of the accident analyses.

3/4.6.1.6 CONTAINMENT STRUCTURAL INTEGRITY

This limitation ensures that the structural integrity of the containment vessel will be maintained comparable to the original design standards for the life of the facility. Structural integrity is required to ensure that the vessel will withstand the maximum pressure of 40.0 psig in the event of a LOCA. The visual and Type A leakage tests are sufficient to demonstrate this capability.

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

3/4.6.2.1 and 3/4.6.2.2 CONTAINMENT QUENCH AND RECIRCULATION SPRAY SYSTEMS

The OPERABILITY of the containment spray systems ensures that containment depressurization and subsequent return to subatmospheric pressure will occur in the event of a LOCA. The pressure reduction and resultant termination of containment leakage are consistent with the assumptions used in the accident analyses.

The recirculation spray system consists of four 50 percent capacity subsystems each composed of a spray pump, associated heat exchanger and flow path. Two of the recirculation spray pumps and motors are located outside containment (RS-P-2A and RS-P-2B) and two pumps and motors are located inside containment (RS-P-1A and RS-P-1B). The flow path from each pump is piped to an individual 180° recirculation spray header inside containment. Train "A" electrical power and river water is supplied to the subsystems containing recirculation spray pumps RS-P-1A and RS-P-2A. Train "B" electrical power and river water is supplied to the subsystems containing recirculation spray pumps RS-P-1B and RS-P-2B.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated November 16, 1992, as supplemented March 11, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 54, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DLCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 11, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 54

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 6-12

3/4 6-13

B 3/4 6-2

B 3/4 6-3

Insert

3/4 6-12

3/4 6-13

B 3/4 6-2

B 3/4 6-3

CONTAINMENT SYSTEMSCONTAINMENT RECIRCULATION SPRAY SYSTEMLIMITING CONDITION FOR OPERATION

3.6.2.2 Four separate and independent containment recirculation spray subsystems, each composed of a spray pump, associated heat exchanger and flow path shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. For subsystems containing recirculation spray pumps 2RSS-P21A or 2RSS-P21B: With one containment recirculation spray subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 72 hours or be in HOT STANDBY within the next 6 hours; restore the inoperable spray subsystem to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the next 30 hours.
- b. For subsystems containing recirculation spray pumps 2RSS-P21C or 2RSS-P21D: See action statements in Specification 3.5.2 or 3.5.3.
- c. For subsystems containing recirculation spray pumps 2RSS-P21A and 2RSS-P21C, or 2RSS-P21B and 2RSS-P21D; apply Action a above if the inoperable subsystem contains recirculation spray pumps 2RSS-P21A (or B), and apply Action b above if the inoperable subsystem contains recirculation spray pumps 2RSS-P21C (or D).

SURVEILLANCE REQUIREMENTS

4.6.2.2 Each containment recirculation spray subsystem shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed or otherwise secured in position, is in its correct position;
- b. When tested pursuant to Specification 4.0.5, manually start each recirculation spray pump and verify the pump shaft rotates;
- c. At least once per 18 months by verifying that on a Containment Pressure-High-High signal, each recirculation spray pump starts automatically after a 628 ± 5 second delay.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (continued)

- d. At least once per 18 months, during shutdown, by verifying, that on recirculation flow, each recirculation spray pump develops a differential pressure of ≥ 112 psid at a flow of ≥ 3500 gpm.
- e. At least once per 18 months during shutdown, by:
 - 1. Cycling each power operated (excluding automatic) valve in the flow path not testable during plant operation, through at least one complete cycle of full travel.
 - 2. Verifying that each automatic valve in the flow path actuates to its correct position on a test signal.*
 - 3. Initiating flow through each Service Water subsystem and its two associated recirculation spray heat exchangers, and verifying a flow rate of at least 11,000 gpm.
- f. At least once per 5 years by performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed.

*The specified 18-month surveillance interval during the first fuel cycle may be extended to coincide with completion of the first refueling outage.

NPF-73
CONTAINMENT SYSTEMS

BASES

3/4.6.1.4 AND 3/4.6.1.5 INTERNAL PRESSURE AND AIR TEMPERATURE
(Continued)

of 45 psig and 2) ensure the containment internal pressure returns subatmospheric within 60 minutes following a LOCA. Additional operating margin is provided if the containment average air temperature is maintained above 100°F as shown on Figure 3.6-1.

The limits on the parameters of Figure 3.6-1 are consistent with the assumptions of the accident analyses.

3/4.6.1.6 CONTAINMENT STRUCTURAL INTEGRITY

This limitation ensures that the structural integrity of the containment vessel will be maintained comparable to the original design standards for the life of the facility. Structural integrity is required to ensure that the vessel will withstand the maximum pressure of 44.7 psig in the event of a LOCA. The visual and Type A leakage tests are sufficient to demonstrate this capability.

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

3/4.6.2.1 and 3/4.6.2.2 CONTAINMENT QUENCH AND RECIRCULATION SPRAY SYSTEMS

The OPERABILITY of the containment spray systems ensures that containment depressurization and subsequent return to subatmospheric pressure will occur in the event of a LOCA. The pressure reduction and resultant termination of containment leakage are consistent with the assumptions used in the accident analyses.

The recirculation spray system consists of four 50 percent capacity subsystems each composed of a spray pump, associated heat exchanger and flow path. All recirculation spray pumps and motors are located outside containment and supply flow to two 360° recirculation spray ring headers located in containment. One spray ring is supplied by the "A" train subsystem containing recirculation spray pump 2RSS-P21A and the "B" train subsystem containing recirculation spray pump 2RSS-P21D with the other spray ring being supplied by the "A" train subsystem containing recirculation spray pump 2RSS-P21C and the "B" train subsystem containing recirculation spray pump 2RSS-P21B. When the water in the refueling water storage tank has reached a predetermined extreme low level, the C and D subsystems are automatically switched to the cold leg recirculation mode of emergency core cooling system operation.

BASES

3/4.6.2.3 CHEMICAL ADDITION SYSTEM

The OPERABILITY of the chemical addition system ensures that sufficient NaOH is added to the containment spray in the event of a LOCA. The limits on NaOH minimum volume and concentration, ensure that 1) the iodine removal efficiency of the spray water is maintained because of the increase in pH value, and 2) corrosion effects on components within containment are minimized. These assumptions are consistent with the iodine removal efficiency assumed in the accident analyses.

3/4.6.3 CONTAINMENT ISOLATION VALVES

The OPERABILITY of the containment isolation valves ensures that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment. Containment isolation within the time limits specified ensures that the release of radioactive material to the environment will be consistent with the assumptions used in the analyses for both a LOCA and major secondary system breaks.

3/4.6.4 COMBUSTIBLE GAS CONTROL

The OPERABILITY of the equipment and systems required for the detection and control of hydrogen gas ensures that this equipment will be available to maintain the hydrogen concentration within containment below its flammable limit during post-LOCA conditions. Either recombiner unit is capable of controlling the expected hydrogen generation associated with 1) zirconium-water reactions, 2) radiolytic decomposition of water, and 3) corrosion of metals within containment. These hydrogen control systems are consistent with the recommendations of Regulatory Guide 1.7, "Control of Combustible Gas Concentrations in Containment Following a LOCA."

3/4.6.5 SUBATMOSPHERIC PRESSURE CONTROL SYSTEM

3/4.6.5.1 STEAM JET AIR EJECTOR

The closure of the manual isolation valves in the suction of the steam jet air ejector ensures that 1) the containment internal pressure may be maintained within its operation limits by the mechanical vacuum pumps and 2) the containment atmosphere is isolated from the outside environment in the event of a LOCA. These valves are required to be closed for containment isolation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 174 TO FACILITY OPERATING LICENSE NO. DPR-66
AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. NPF-73

DUQUESNE LIGHT COMPANY
OHIO EDISON COMPANY
PENNSYLVANIA POWER COMPANY
THE CLEVELAND ELECTRIC ILLUMINATING COMPANY
THE TOLEDO EDISON COMPANY

BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-334 AND 50-412

1.0 INTRODUCTION

By letters dated November 16, 1992, and March 11, 1993, Duquesne Light Company, (the licensee) for Beaver Valley Power Station, Units 1 and 2, proposed modifications to the station's Technical Specifications, Section 3.6.2.2, containment recirculation spray system. The supplemental letter dated March 11, 1993, did not change the initial proposed no significant hazards consideration determination. The proposed change in TS 3.6.2.2 for Beaver Valley Unit 1 will add a new Action Statement b. to address the inoperability of two (2) subsystems. Additionally, the expired portions of Figure 3.6-1 which was applicable for Cycle 8, will be removed. The proposed change in TS 3.6.2.2 for Beaver Valley Unit 2 will add a new Action Statement c. to address the inoperability of two (2) subsystems in the same train. Also, for Beaver Valley Unit 2, the Surveillance Requirements 4.6.2.2.d and e were shifted to page 3/4 6-13 during typing of page 3/4 6-12.

2.0 EVALUATION

The proposed changes for Beaver Valley Unit 1 Technical Specification 3.6.2.2 will be the addition of Action Statement b. Action Statement b. requires restoring at least one inoperable subsystem to operable status within 72 hours or placing the plant in hot standby within the next 6 hours and in cold shutdown within the following 30 hours. This action requirement is consistent with the design of the system described in Updated Final Safety Analysis Report (UFSAR) Section 6.4 and does not affect the accident analyses described in Section 14 since with two subsystems inoperable the remaining two subsystems are available to supply 100 percent of the required flow.

The proposed changes for Beaver Valley Unit 2 Technical Specification 3.6.2.2 will be the correction of a misspelled word in Action Statement a., the

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addition of Action Statement c., and the addition of Bases for TS 3.6.2.2 Action Statement c., explaining the function of Action Statement c.

Action Statement c. provides for two (2) inoperable subsystems in the same train and requires the application of Action Statement a. to inoperable subsystem A or B and the application of Action b. to inoperable subsystem C or D. This action requirement is consistent with the design of the system described in UFSAR Section 6.2 and does not affect the accident analyses described in Section 15 since with two (2) subsystems inoperable the remaining two subsystems are available to supply 100 percent of the required flow.

The proposed change for Beaver Valley Power Station, Unit 1 TS 3.6.2.2 is below:

- b. With two (2) containment recirculation spray subsystems inoperable restore at least one inoperable subsystem to OPERABLE status within 72 hours or be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

The proposed changes for Beaver Valley Power Station, Unit 2 TS 3.6.2.2 are below:

- a. For subsystems containing recirculation spray pumps 2RSS-P21A or 2RSS-P21B: with one containment recirculation spray subsystem inoperable restore the inoperable subsystem to OPERABLE status within 72 hours or be in HOT STANDBY within the next 6 hours; restore the inoperable spray subsystem to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the next 30 hours.
- c. For subsystems containing recirculation spray pumps 2RSS-P21A and 2RSS-P21C, or 2RSS-P21B and 2RSS-P21D; apply Action a above if the inoperable subsystem contains recirculation spray pumps 2RSS-P21A (or B), and apply Action b above if the inoperable subsystem contains recirculation spray pumps 2RSS-P21C (or D).

The changes are consistent with the UFSAR system description, the accident analyses and the Standard Technical Specifications since continued operation is allowed only as long as a combination of recirculation spray subsystems is operable which gives 100 percent of the flow required in the safety analyses and the difference between those subsystems which are capable of switching to the cold leg recirculation mode of emergency core cooling system operation and those that are not is accounted for in the action statements. This is clearly explained in the bases provided by the licensee.

The proposed changes ensure the continued function of the recirculation spray system (RSS) to reduce the containment temperature and return the containment pressure to subatmospheric following a break in either the primary or secondary system piping inside containment with one or two RSS subsystems inoperable. Therefore, the staff finds the proposed changes acceptable.

Based on the above evaluation, the staff has determined that the licensee's proposed changes to Beaver Valley Power Station, Units 1 and 2, TS 3.6.2.2, containment recirculation spray systems, and associated TS Bases, containment systems, are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (58 FR 5430). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Narvaez L. Stinson

Date: August 11, 1993