



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 2, 1995

Mr. J. E. Cross
Senior Vice President and
Chief Nuclear Officer
Nuclear Power Division
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

SUBJECT: ENVIRONMENTAL ASSESSMENT OF REQUEST FOR EXEMPTION FROM PARAGRAPH
III.D.2(b)(ii) OF 10 CFR PART 50, APPENDIX J - BEAVER VALLEY POWER
STATION, UNIT NOS. 1 AND 2 (TAC NOS. M84223 AND 84224)

Dear Mr. Cross:

Enclosed is a copy of the Environmental Assessment and Finding of No
Significant Impact related to your application for exemption dated February 4,
1994. The proposed exemption would allow you to substitute local leak rate
testing for an overall air lock leakage test where the design permits.

The assessment is being forwarded to the Office of the Federal Register for
publication.

Sincerely,

Donald S. Brinkman, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-334
and 50-412

Enclosure: Environmental Assessment

cc w/encl: See next page

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J. E. Cross
Duquesne Light Company

Beaver Valley Power Station
Units 1 & 2

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NOS. 50-334 AND 50-412

BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2

ENVIRONMENTAL ASSESSMENT AND FINDING OF

NO SIGNIFICANT IMPACT

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from certain requirements of its regulations to Facility Operating License Nos. DPR-66 and NPF-73, issued to Duquesne Light Company et al. (the licensee), for operation of the Beaver Valley Power Station, Unit Nos. 1 and 2, located in Beaver County, Pennsylvania.

ENVIRONMENTAL ASSESSMENT

Identification of the Proposed Action:

The proposed action is in accordance with the licensee's application dated February 4, 1994, for exemption from certain requirements of paragraph III.D.2(b)(ii) of 10 CFR Part 50, Appendix J. The proposed exemption would allow substitution of local leak rate testing (where the design permits) in lieu of an overall air lock leakage test which would otherwise be required after performing maintenance on the air lock. The air lock components for which this exemption would be applicable would be those where the design of the affected component(s) would permit local leak testing at a pressure of

not less than Pa (the calculated peak containment internal pressure related to the design basis accident and specified either in the technical specification or associated bases). The use of the words "where the design permits" is intended to require that two criteria be satisfied if the proposed exemption is applied. The first criterion, is that any component which has had maintenance performed on it have local leak rate test provisions included into its design. The second criterion is that the method for measuring the component's local leak rate must be equivalent to or more conservative than the method which would be used on that component during performance of an overall air lock leakage test.

The Need for the Proposed Action:

Paragraph III.D.2.(b)(ii) of 10 CFR Part 50, Appendix J, requires licensees to perform an overall air lock leak test at Pa at the end of periods during which the air lock has been opened when containment integrity was not required. Performance of an overall air lock leak test requires 4 to 6 hours and results in additional occupational radiation exposures. The time required to perform overall tests at the conclusion of a plant shutdown can result in delaying plant restart. Application of the proposed exemption would be applicable only to those air lock components provided with local leak rate testing capabilities and for which the leak rate does not exceed the leak rate that has been measured on that component during performance of previous acceptable overall air lock leakage tests. Therefore, local leak rate tests provide adequate assurance that the offsite doses following a design basis accident will be within acceptable limits.

Environmental Impacts of the Proposed Action:

The Commission has completed its evaluation of the licensee's application. The proposed exemption will not increase the probability or consequences of accidents. The probability of accidents is not increased because the air locks do not affect the initiation of any design basis accident. The consequences of an accident are not increased because the component local leak rates will not be permitted to exceed the leak rate which would be measured on that component during performance of the overall air lock leakage test. No changes are being made in the types of any radioactive effluents that may be released offsite as a result of the proposed exemption, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does involve features located entirely within the restricted area as defined in 10 CFR Part 20. It does not effect nonradiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed action.

Alternatives to the Proposed Action:

Since the Commission has concluded there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial

of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources:

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the Beaver Valley Power Station Units Nos. 1 and 2.

Agencies and Persons Consulted:

In accordance with its stated policy, on May 9, 1995, the staff consulted with the Pennsylvania State official, Robert C. Maiers of the Bureau of Radiation Protection, Department of Environmental Resources, regarding the environmental impact of the proposed action. The State official had no comments.

FINDING OF NO SIGNIFICANT IMPACT

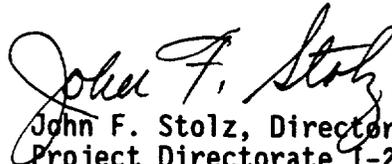
Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated February 4, 1994, which is available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120

L Street, NW., Washington, DC, and at the local public document room located at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001.

Dated at Rockville, Maryland, this 2nd day of June 1995.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "John F. Stolz".

John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

June 2, 1995

Mr. J. E. Cross
Senior Vice President and
Chief Nuclear Officer
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Sincerely,

Original signed by:

Donald S. Brinkman, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-334
and 50-412

Enclosure: Environmental Assessment

cc w/encl: See next page

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JZwolinski ACRS (4)
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NAME	MO'Brien	DBrinkman ^{DAB}	R Weisman ^{No legal objection with comments}	JStolz	
DATE	5/17/95	5/17/95	5/30/95	5/18/95	

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