

April 1, 1993

Docket Nos. 50-334
and 50-412

Mr. J. D. Sieber, Senior
Vice President and Chief
Nuclear Officer
Nuclear Power Division
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077-0004

Dear Mr. Sieber:

SUBJECT: BEAVER VALLEY POWER STATION, UNITS 1 AND 2 - FEDERAL REGISTER NOTICE
RELATED TO TECHNICAL SPECIFICATION CHANGE REQUEST NOS. 205 AND 71
(TAC NOS. M85081 AND 85082)

The Commission has forwarded the enclosed "Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing" to the Office of the Federal Register for publication.

The notice relates to your December 30, 1992, application to amend the Technical Specifications and Bases to allow sleeving at the steam generator tube support plate and tubesheet regions in accordance with processes performed by the vendors Babcock & Wilcox and Westinghouse.

Sincerely,
Original signed by
Gordon E. Edison, Sr. Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure
see next page

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Mr. J. D. Sieber
Duquesne Light Company

Beaver Valley Power Station
Units 1 & 2

cc:

Jay E. Silberg, Esquire
Shaw, Pittman, Potts & Trowbridge
2300 N Street, NW.
Washington, DC 20037

Nelson Tonet, Manager
Nuclear Safety
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077

Commissioner Roy M. Smith
West Virginia Department of Labor
Building 3, Room 319
Capitol Complex
Charleston, West Virginia 25305

John D. Borrows
Director, Utilities Department
Public Utilities Commission
180 East Broad Street
Columbus, Ohio 43266-0573

Director, Pennsylvania Emergency
Management Agency
Post Office Box 3321
Harrisburg, Pennsylvania 17105-3321

Ohio EPA-DERR
ATTN: Zack A. Clayton
Post Office Box 1049
Columbus, Ohio 43266-0149

Bureau of Radiation Protection
Pennsylvania Department of
Environmental Resources
ATTN: R. Barkanic
Post Office Box 2063
Harrisburg, Pennsylvania 17120

Mayor of the Borough of
Shippingport
Post Office Box 3
Shippingport, Pennsylvania 15077

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 181
Shippingport, Pennsylvania 15077



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

April 1, 1993

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and 50-412

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Vice President and Chief
Nuclear Officer
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The notice relates to your December 30, 1992, application to amend the Technical Specifications and Bases to allow sleeving at the steam generator tube support plate and tubesheet regions in accordance with processes performed by the vendors Babcock & Wilcox and Westinghouse.

Sincerely,

A handwritten signature in cursive script that reads "G E Edison".

Gordon E. Edison, Sr. Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure
see next page

ENCLOSURE

UNITED STATES NUCLEAR REGULATORY COMMISSION

DUQUESNE LIGHT COMPANY

DOCKET NOS. 50-334 and 50-412

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO
FACILITY OPERATING LICENSES, PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR A HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. DPR-66 and NPF-73 issued to Duquesne Light Company (the licensee) for operation of the Beaver Valley Power Station, Unit Nos. 1 and 2 located in Shippingport, Pennsylvania.

The proposed amendments would modify the Technical Specifications and Bases to allow sleeving at the steam generator tube support plate and tubesheet regions in accordance with processes performed by the vendors, Babcock & Wilcox and Westinghouse.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant

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reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

The following evaluation is provided for the no significant hazards consideration standards.

1. Does the change involve a significant increase in the probability or consequences of an accident previously evaluated?

Some steam generator tubes have been found to have a varying amount of wall degradation. When the degradation is extensive, the normal practice of plugging defective tubes reduces the effectiveness of the steam generators and eventually will reduce the performance of the nuclear steam supply system. An alternative to plugging tubes is installing a sleeve as a new pressure boundary inside the original tube to bridge the degraded area, thus permitting the tubes to remain in service. The integrity of the repaired steam generator tubes will be equivalent to that of the original tube and will allow the tube to continue performing its heat transfer function.

The proposed change allows the installation of steam generator tube sleeves in accordance with the vendor methodologies provided by the B&W Kinetic welded sleeving process described in NRC approved topical report BAW-2094P, Revision 1 and the Westinghouse laser welded sleeving process described in WCAP-13483, Revision 1, provided for NRC review and acceptance. The sleeve installation process procedures described in BAW-2094P, Revision 1 will be revised to include the kinetic sleeve "tooling" and installation process parameter changes described in NRC approved BAW-2045PA, Revision 1, January 1992, "Recirculating Steam Generators Kinetic Qualification for 3/4 Inch OD Tubes." These changes were incorporated to resolve field problems or to improve the sleeve installation rate and will not alter the basic installed configuration of the sleeve as described in BAW-2094P, Revision 1. We have reviewed the methodology described in BAW-2094P, Revision 1 and determined that they are applicable to the Beaver Valley units and provide a safe and efficient alternative to plugging. We have reviewed the methodologies described in these vendor reports and determined that they provide a safe and efficient alternative to plugging. Eddy current techniques are available to perform sleeve and tube inspections for defect detection and to verify proper installation of the sleeve. Available techniques are capable of providing adequate defect sensitivity in the required areas of the tube and sleeve pressure boundary. Proprietary methods described in the vendor reports with

supporting qualification data demonstrate the inspectability of the sleeve and underlying tube. In addition, we are committing to qualify the adequacy of any system that is used for periodic inservice inspection and to evaluate and, if practical, implement testing methods as better methods are developed and qualified for use.

The structural integrity of the repaired tube is restored to that of an undegraded tube and the tube and sleeves will be inspected periodically in accordance with the technical specification surveillance requirements. Sleeving does not affect the UFSAR steam generator tube rupture accident, therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the change create the possibility of a new or different kind of accident from [any] accident previously evaluated?

Both the structural integrity and the heat transfer capability of the steam generators will not be significantly affected by the installation of sleeves. In addition, the sleeves are attached to the inside of the tubes and cannot interact with any of the other plant systems. The sleeves have been analyzed and tested and the repair methods have been evaluated to ensure they satisfy the required design conditions. Sleeving returns the degraded tube to a serviceable condition and the sleeved tube functions in essentially the same manner as the original tube. Therefore, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Does the change involve a significant reduction in a margin of safety?

The heat transfer capabilities of the steam generators will be improved by utilizing the sleeving process rather than the currently required plugging. Installing sleeves slightly reduces the RCS flow and heat transfer capabilities, however, this reduction is significantly less than that of tubes that have been plugged. Sleeving maintains the structural integrity of the steam generators to ensure the RCS pressure boundary is adequate for the expected design conditions, therefore, the proposed change does not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied.

Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room P-223, Phillips Building, 7920 Norfolk Avenue, Bethesda, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may

be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By May 6, 1993 , the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the local public document room located at the local public document room located at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The

petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact.

Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555, by the above date. Where petitions are filed during the last 10 days of the notice period, it is

requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to Walter R. Butler: petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Gerald Charnoff, Esquire, Jay E. Silberg, Esquire, Shaw, Pittman, Potts & Trowbridge, 2300 N Street, NW., Washington, DC 20037, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated December 30, 1992, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the local public document room located at the

B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania
15001.

Dated at Rockville, Maryland, this 1st day of April 1993.

FOR THE NUCLEAR REGULATORY COMMISSION



Gordon E. Edison, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation