

December 31, 1992

Docket No. 50-334
Serial No. BV-92-042

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Mr. J. D. Sieber, Vice President
Nuclear Group
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077-0004

Dear Mr. Sieber:

SUBJECT: AMENDMENT NO. 168 TO FACILITY OPERATING LICENSE DPR-66:
LICENSE CONDITION EXTENDING CERTAIN SURVEILLANCE TEST PERFORMANCE
INTERVALS - CHANGE REQUEST NO. 198 (TAC NO. M84229)

The Commission has issued the enclosed Amendment No. 168 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1, in response to your application dated August 18, 1992, as modified by letter dated September 2, 1992. This amendment adds License Condition 2.C(9) which provides for limited extension of the performance interval for certain surveillance tests to coincide with the completion of the 9th refueling outage.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Albert W. De Agazio, Sr. Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 168 to DPR-66
2. Safety Evaluation

cc w/enclosures:
See next page

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DE: EMCB
J. Strosnider 12/13/92

EMCB

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OFFICE	LA:PDI-4	PM:PDI-4 <i>mlc</i>	D:PDI-4	OGC <i>MZ NW</i>	OTSB <i>CG</i>
NAME	SNorris	ADeAgazio:cn	JSto12	MZOBLET	CIGrimes
DATE	12/7/92	12/17/92	12/17/92	12/29/92	12/18/92

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subject to
minor change
in red ink

No policy or precedent
only one cycle.

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PDR ADOCK 05000334
P PDR

DF01

Mr. J. D. Sieber
Duquesne Light Company

Beaver Valley Power Station
Units 1 & 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.168
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated August 18, 1992, as modified by letter dated September 2, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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P PDR

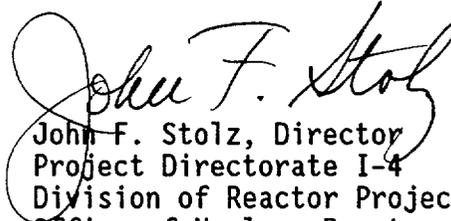
2. Accordingly, Facility Operating License No. DPR-66 is hereby amended by the addition of License Condition 2.C(9) to state:*

(9) Surveillance Interval Extension

The performance interval for those surveillance requirements identified in the licensee's request for surveillance interval extension dated August 18, 1992, as modified by letter dated September 2, 1992, shall be extended to 24 months to coincide with Cycle 9 refueling outage.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Page 6 of the license

Date of Issuance: December 31, 1992

*Page 6 is attached, for convenience, for the composite license to reflect this change.

(6) Systems Integrity

Duquesne Light Company shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
2. Integrated leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

(7) Iodine Monitoring

Duquesne Light Company shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel,
2. Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

(8) Backup Method for Determining Subcooling Margin

Duquesne Light Company shall implement a program which will ensure the capability to accurately monitor the Reactor Coolant System subcooling margin. This program shall include the following:

1. Training of personnel, and
2. Procedures for monitoring.

(9) Surveillance Interval Extension

The performance interval for those surveillance requirements identified in the licensee's request for surveillance interval extension dated August 18, 1992, as modified by letter dated September 2, 1992, shall be extended to 24 months to coincide with the Cycle 9 refueling outage.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 168 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

1.0 INTRODUCTION

By application August 18, 1992, as amended by letter of September 2, 1992, Duquesne Light Company (DLC) proposed a change to Facility Operating License No. DPR-66. Specifically, the proposed change would add License Condition 2.C(9) that would allow extension of the performance interval for certain surveillance requirements. The proposed extensions would allow the identified surveillance testing to coincide with the completion of the 9th refueling outage (9 RFO), but the extended intervals are not to exceed 24 months.

2.0 DISCUSSION

The design length for Cycle 9 was 514 effective full power days which represents about 20 months of power operation with a capacity factor of 85%. The cycle was originally planned to start in June 1991 and last until February 1993. Because of delays in startup from the refueling outage followed by other unscheduled outages, 9 RFO has been rescheduled to begin April 12, 1993. This revised outage date results in the expiration of certain surveillance test intervals prior to the outage. The proposed license condition would provide for extension of the intervals until the outage. Without the extensions, an otherwise unnecessary plant shutdown would be required.

In its application for amendment, DLC has asserted that the reliability defined by the normal surveillance interval would not be significantly reduced by the extensions. This assertion is based upon the following considerations:

- Present monitoring of instrumentation and ongoing technical specification surveillance tests provide assurance that the equipment involved in the extended surveillance tests will remain in an operable condition until testing is performed at the next refueling outage.

- Periodic surveillance tests have been performed since the last refueling outage to monitor system and component performance and to detect any system degradation. Surveillance testing will be performed during the requested extension interval providing added assurance that the reliability of equipment associated with the extended surveillance will not be degraded significantly by the proposed limited-time extension.
- The electronic components in the reactor protection system and engineered safety features actuation system have shown a very high degree of reliability.
- Section XI of the ASME Code defines a refueling interval as 18 months but no more than 24 months. Based on this definition, all IST testing required at refueling intervals does not have to be performed until the unit is shutdown for refueling.

The surveillance affected by this request for interval extensions are identified as follows:

Technical Specification

Surveillance Test

4.3.1.1.1 Table 4.3-1

Reactor Trip System Instrumentation
Channel Calibration and Functional
Test

4.3.1.1.3 Table 3.3-2

Reactor Trip System Instrumentation
Response Times

4.3.2.1.1 Table 4.3-2

Engineered Safety Features Actuation
System Instrumentation
Channel Calibration and Functional
Test

4.3.2.1.3 Table 3.3-5

Engineered Safety Features Actuation
System Instrumentation
Response Times

4.3.3.1 Table 4.3-3

Radiation Monitoring Instrumentation
Channel Calibration

4.3.3.3.1 Table 4.3-4

Seismic Monitoring Instrumentation
Channel Calibration

4.3.3.5 Table 4.3-6

Remote Shutdown Instrumentation
Channel Calibration

Technical Specification

Surveillance Test

4.3.3.8 Table 4.3-7

Accident Monitoring Instrumentation
Channel Calibration

4.4.5.3

Steam Generator Category C-3
Inspection

4.4.9.3.1.b

Overpressure Protection System
Channel Calibration

4.4.11.1.b

PORV Operability
Channel Calibration

4.4.11.3

Valve Exercise

4.5.1.2.b

Accumulator Operability
Channel Calibration

4.5.2.f

Valve Exercise

4.5.3.1

Valve Exercise

4.6.3.1.2.e

Containment Isolation Valves
Check Valve Lift Test

4.6.3.1.2.f

Valve Exercise

4.7.3.1.c

Valve Exercise

4.7.13.1.b

Auxiliary River Water Test

4.8.1.1.1.b and 4.8.1.2

Emergency Switchgear Operation and
Diesel Generator Auto Load Tests

4.8.1.1.2.b

Diesel Generator Test and Inspection

4.8.2.3.2

DC Distribution
Battery Cell Conditions and Charger
Capacity

4.8.2.4.2

DC Distribution
Battery and Charger Service Testing

3.0 EVALUATION

The delays experienced following the last refueling outage and subsequent unscheduled outages have resulted in the need to reschedule the next refueling outage so as to obtain optimum fuel burnup prior to refueling. Thus, the start of the next refueling outage has been moved from early February 1993 to April 12, 1993. The impact of this rescheduling is that certain surveillance tests that are performed during refueling will fall due (including the extensions permitted by Technical Specification 4.0.2) before the start of the outage, unless the surveillance intervals are extended.

Periodic surveillance requirements were not intended to affect adversely safe plant operations simply because a specified surveillance interval does not coincide with plant operating schedules. Normally, variations in those schedules can be accommodated through the existing technical specifications. Specifically, Technical Specification 4.0.2 is an administrative control which ensures that surveillance tests are performed within the specified interval, but it provides for an allowable tolerance for performing surveillance beyond the nominal surveillance interval. This tolerance provides operational flexibility to allow for scheduling and performance considerations while still ensuring that the reliability of the equipment or system associated with the surveillance is not significantly degraded beyond that obtained from the nominal specified surveillance interval. However, circumstances can develop wherein the relief provided by Technical Specification 4.0.2 is inadequate, but good cause for additional relief can be demonstrated by the licensee.

Such is the case here. In this instance, DLC has provided compelling evidence that the change in the refueling schedule was not undertaken for a reason or in a manner adverse to safety, that reasonable assurance exists that equipment associated with the subject surveillance will not be degraded significantly by the requested interval extensions, and that good cause exists for granting the extensions. The surveillance interval extensions proposed by DLC would result in a slightly diminished confidence in the reliability that would be provided by Technical Specification 4.0.2. The proposed license condition would extend the allowable surveillance intervals from $22\frac{1}{2}$ months (nominal 18 months plus $4\frac{1}{2}$ months allowable extension per 4.0.2) to a maximum of 24 months. The staff believes that the additional $1\frac{1}{2}$ months extension is not significant, therefore, the staff finds the proposed license condition acceptable.

DLC requested extension of the due date for the Category C-3 Inspection of steam generator tubes specified by Technical Specification 4.4.5.3.b. This inspection, when required, is to be conducted at least once per 20 months. The provisions of Technical Specification 4.0.2 are not noted as being inapplicable, and therefore DLC may extend the surveillance interval from the nominal 20 months to 24 months, consistent with Technical Specification 4.4.5.3.a. The Category C-3 inspection is due January 11, 1993; thus, under the provisions of Technical Specification 4.0.2 the Category C-3 inspection could be conducted as late as May 11, 1993.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 47128). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Albert W. De Agazio

Date: December 31, 1992