

September 20, 1990

Docket No. 50-334
Serial No. BV-90-013

Mr. J. D. Sieber, Vice President
Nuclear Group
Duquesne Light Company
P. O. Box 4
Shippingport, Pennsylvania 15007

Dear Mr. Sieber:

SUBJECT: CORRECTION TO AMENDMENT 155 TO FACILITY OPERATING LICENSE DPR-66:
REACTOR TRIP SYSTEM AND ENGINEERED SAFETY FEATURES ACTUATION SYSTEM
TRIP SETPOINTS (TAC 71869)

On August 17, 1990, the Commission issued Amendment No. 155 to Facility Operating License DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consisted of changes to the Appendix A Technical Specifications (TSs) in response to your application dated March 2, 1989, as supplemented by letter dated June 12, 1990. The amendment revised certain reactor trip system and engineered safety features actuation system instrumentation trip setpoints in TS Tables 2.2-1 and 3.3-4.

One of the changes made to Table 3.3-4 inadvertently changed the values for Item 1.1 d, Refueling Water Storage Tank Level - Auto QS Flow Reduction. While your application identified above requested the change that was made, that request had been superseded by a later application, submitted June 22, 1989, but which was incorporated into the TSs by Amendment 143 on July 27, 1989. Therefore, you should substitute the enclosed corrected page 3/4 3-22a into your TSs. To avoid confusion, this corrected page continues to bear the "Amendment 155" notation.

Also included is a corrected Page 4 to the Safety Evaluation supporting Amendment 155 which corrects a typographical error in the issue date.

Sincerely,

original signed by Albert De Agazio

Albert W. De Agazio, Sr. Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

PDI-4:LA
SNorris
9/13/90

PDI-4:PM
ADeAgazio:dr
9/17/90

PDI-4:D
DStolz
9/20/90

[71869 CORRECTION]

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Also included is a corrected Page 4 to the Safety Evaluation supporting Amendment 155 which corrects a typographical error in the issue date.

Sincerely,

A handwritten signature in cursive script, reading "Albert W. De Agazio, Sr.".

Albert W. De Agazio, Sr. Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

Mr. J. Sieber
Duquesne Light Company
cc:

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Beaver Valley Power Station
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BEAVER VALLEY - UNIT 1

3/4 3-22a

Amendment No. 73, 28, 143

JUL 27 1989

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
1.1 SAFETY INJECTION-TRANSFER FROM INJECTION TO THE RECIRCULATION MODE		
a. Manual Initiation	Not Applicable	Not Applicable
b. Automatic Actuation Logic Coincident with Safety Injection Signal	Not Applicable	Not Applicable
c. Refueling Water Storage Tank Level-Low	18'8-1/2"	≥18'2-1/2" and ≤19'2-1/2"
d. Refueling Water Storage Tank Level - Auto QS Flow Reduction	8'6"	≥8'0" and ≤9'0"

An analysis has been performed to update the calculation of these instrument inaccuracies using a more current methodology for determining the required trip setting and allowable value limits. The methodology used is described in detail in Westinghouse WCAP-11419. The proposed changes to Table 2.2-1 and Table 3.3-4 revise the allowable values based on the calculation of the instrument inaccuracies using a more current methodology.

By using the methodology described in Westinghouse WCAP-11419, the plant gains added operational flexibility and yet remains within the analytical limit values accounted for in the various accident analysis. In addition, the methodology allows for a sensor drift factor and an increased rack drift factor. The proposed changes to revise the allowable values in Table 2.2-1 and Table 3.3-4 of the Beaver Valley Unit 1 Technical Specifications are based on the calculation of the instrument accuracies by using approved current methodology and are acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The staff has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 17, 1990

Principal Contributor: S. Rhow

DATED:

AMENDMENT NO. TO FACILITY OPERATING LICENSE NO. -

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