

July 5, 1985

Docket No. 50-334

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Mr. J. J. Carey, Vice President
Nuclear Division
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

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Dear Mr. Carey:

SUBJECT: ISSUANCE OF AMENDMENT (LICENSING ACTION TAC NO. 57289)

The Commission has issued the enclosed Amendment No. 94 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated March 21, 1985.

The amendment changes the Technical Specifications for Beaver Valley Unit No. 1 to clarify the reactor plant component cooling pump and river water pump surveillance requirements. The new surveillance requirements specify that each pump develops the required differential pressure and flow rate when tested in accordance with Specification 4.0.5, which in turn requires that certain pumps and valves be tested in accordance with ASME Code Section XI.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/PSTam

Peter S. Tam, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:

1. Amendment No. 94 to DPR-66
2. Safety Evaluation

cc w/enclosures:
See next page

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Glatmas
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 5, 1985

Docket No. 50-334

Mr. J. J. Carey, Vice President
Nuclear Group
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

Dear Mr. Carey:

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A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Peter S. Tam".

Peter S. Tam, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:

1. Amendment No. 94 to DPR-66
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. J. J. Carey
Duquesne Light Company

Beaver Valley 1 Power Station

cc:

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State Clearinghouse
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Harrisburg, Pennsylvania 17105

Mr. Jess T. Shumate, Commissioner
State of West Virginia Department
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1900 Washington Street, East
Charleston, West Virginia 25305

Duquesne Light Company

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Beaver Valley 1 Power Station

cc:

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State Department of Health
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Charleston, West Virginia 25305

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
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Mr. Thomas M. Gerusky, Director
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Pennsylvania Department of
Environmental Resources
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Harrisburg, Pennsylvania 17120



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 94
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated March 21, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

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P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 94, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective on issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 5, 1985

ATTACHMENT TO LICENSE AMENDMENT
AMENDMENT NO. 94 TO FACILITY OPERATING LICENSE NO. DPR-66
DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages

3/4 7-12
3/4 7-13

Insert Pages

3/4 7-12
3/4 7-13

PLANT SYSTEMS

3/4.7.3 COMPONENT COOLING WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.3.1 At least two component cooling water subsystems shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With less than two component cooling water subsystems OPERABLE, restore at least two subsystems to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.3.1 At least two component cooling water subsystems shall be demonstrated OPERABLE.

- a. At least once per 31 days on a STAGGERED TEST BASIS by:
 1. Verifying that each pump develops the required differential pressure and flow rate when tested in accordance with the requirements of Section 4.0.5.
 2. Cycling each testable power operated or automatic valve servicing safety related equipment through at least one complete cycle of full travel.
 3. Verifying that each valve (manual, power operated or automatic) servicing safety related equipment that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. At least once per 18 months during shutdown, by cycling each power operated valve servicing safety related equipment that is not testable during plant operation, through at least one complete cycle of full travel.

PLANT SYSTEMS

3/4.7.4 REACTOR PLANT RIVER WATER SYSTEM (RPRWS)

LIMITING CONDITION FOR OPERATION

3.7.4.1 At least two reactor plant river water subsystems supplying safety-related equipment shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With less than two RPRWS subsystems OPERABLE, restore at least two subsystems to OPERABLE status within 72 hours or be in at least HOT STANDBY within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.4.1 At least two RPRW subsystems shall be demonstrated OPERABLE:

- a. At least once per 31 days on a STAGGERED TEST BASIS by:
 1. Verifying that each pump develops the required differential pressure and flow rate when tested in accordance with the requirements of Section 4.0.5.
 2. Cycling each testable power operated or automatic valve servicing safety related equipment through at least one complete cycle of full travel.
 3. Verifying that each valve (manual, power operated or automatic) servicing safety related equipment that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. At least once per 18 months during shutdown, by cycling each power operated valve servicing safety related equipment that is not testable during plant operation, through at least one complete cycle of full travel.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 94 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

INTRODUCTION

By letter dated March 21, 1985, Duquesne Light Company (the licensee) submitted a proposed amendment to the Technical Specifications (Appendix A of Operating License No. DPR-66) for Beaver Valley Power Station, Unit No. 1, to clarify the surveillance requirements for the Component Cooling Water and Reactor Plant River Water Pumps.

EVALUATION AND DISCUSSION

Currently, Surveillance Requirements 4.7.3.1.a.1, 2, and 3, and 4.7.4.1.a.1, 2, and 3 for the Component Cooling Water Pumps and Reactor Plant River Water Pumps, respectively, specify pump testing requirements. Surveillance Requirement 4.0.5 specifies the ASME Boiler and Pressure Vessel Code, Section XI, for inservice testing of pumps. The licensee proposes to delete the existing specifications for pump testing in Sections 4.7.3 and 4.7.4 and to substitute a reference to the section 4.0.5 requirements. The licensee states that the existing section 4.7.3 and 4.7.4 pump surveillance requirements are inconsistent with the ASME Section XI pump testing requirements specified by section 4.0.5. As the ASME Section XI requirements provide for a complete, acceptable method of pump testing, the licensee proposes to utilize solely these requirements and eliminate the inconsistency.

We have evaluated the proposed changes to the Technical Specifications and conclude that these changes are administrative and do not involve any physical change to the plant's safety-related structures, systems or components. Further, these changes do not increase the likelihood of a malfunction of safety-related equipment, or increase the consequences of an accident previously analyzed or create the possibility of a malfunction different from those previously evaluated. Therefore, we find the licensee's requested changes to be acceptable.

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ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 5, 1985

Principal Contributor:

Glenn W. Meyer